

# NIST Neutron Source Preconceptual Design

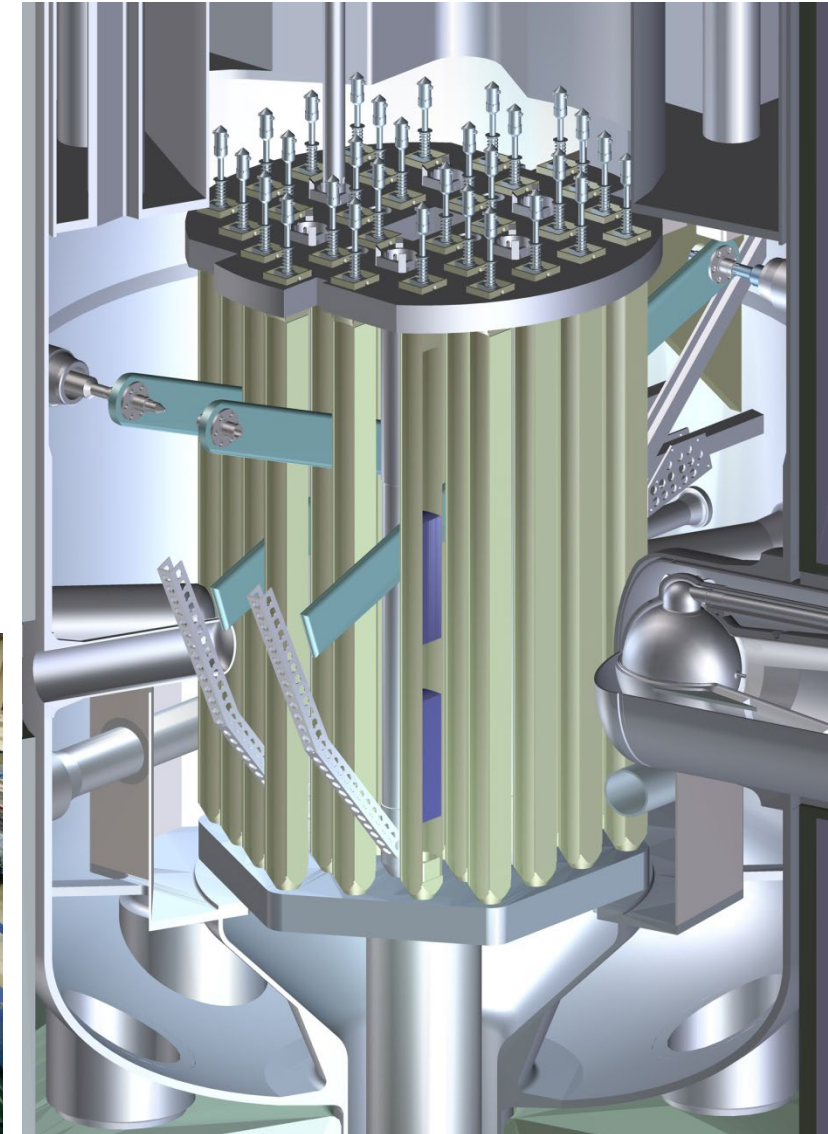
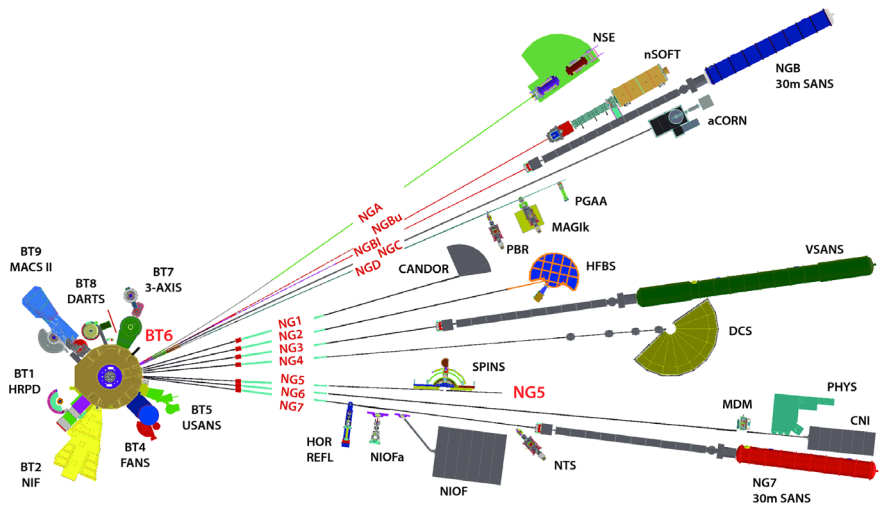
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## Disclaimer

*Certain commercial equipment, instruments, or materials are identified in this study in order to specify the experimental procedure adequately. Such identification is not intended to imply recommendation or endorsement by the National Institute of Standards and Technology, nor is it intended to imply that the materials or equipment identified are necessarily the best available for the purpose.*

# Introduction: NCNR & NBSR

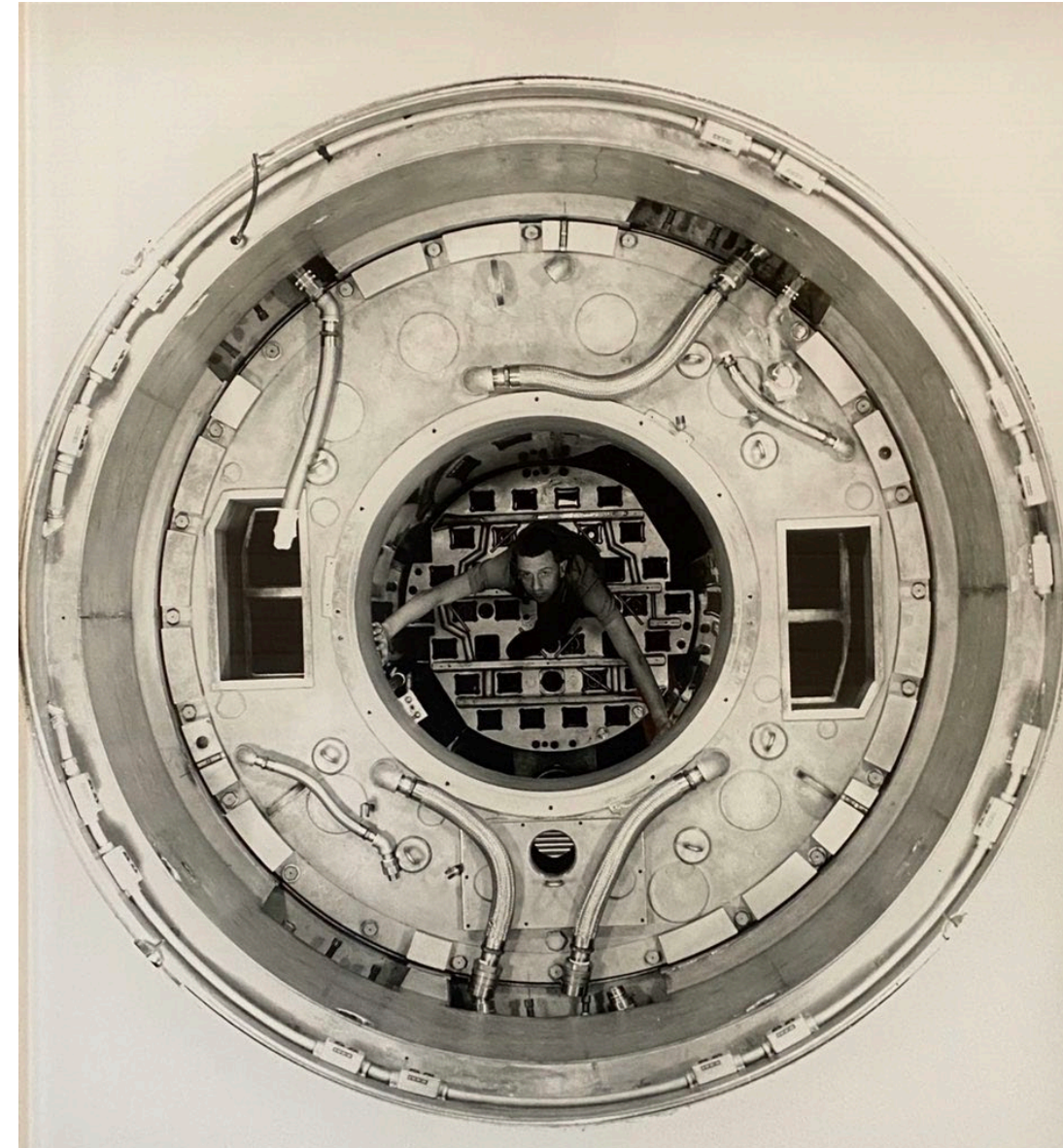
- NCNR is one of the USA's primary resources for neutron research
- NBSR history of successful operation since 1967
- NBSR license to expire in 2029, extension is planned
- Replacement => **NIST Neutron Source (NNS)** is conceptualized.
- Neutronics, thermal-hydraulic, beam delivery, and facilities design activities are ongoing.





# The NBSR Core Characteristics

- HEU fuel (93%  $^{235}\text{U}$ ) –  $\text{U}_3\text{O}_8 + \text{Al}$ 
  - 350 g  $^{235}\text{U}$ /fuel element
  - 34 plates: 11"×2.5"×0.02"
  - Heavy water coolant ( $\text{D}_2\text{O}$ )
- 30 Fuel Elements
  - ~38-day fuel cycle at 20 MW
  - 4 fresh elements per cycle (usually) with a management scheme to shuffle other elements in the core
  - ~960 g of  $^{235}\text{U}$  consumed per cycle
  - 7 or 8 cycles per element depending on where it is loaded initially
- Axially split-core, Argonne CP-5 class
  - Material Test Reactor
  - BTs and CNS(s) are at the thermal neutron flux trap elevation
  - BT fluxes  $\sim 1.5 \times 10^{14}$  n/cm<sup>2</sup>-s



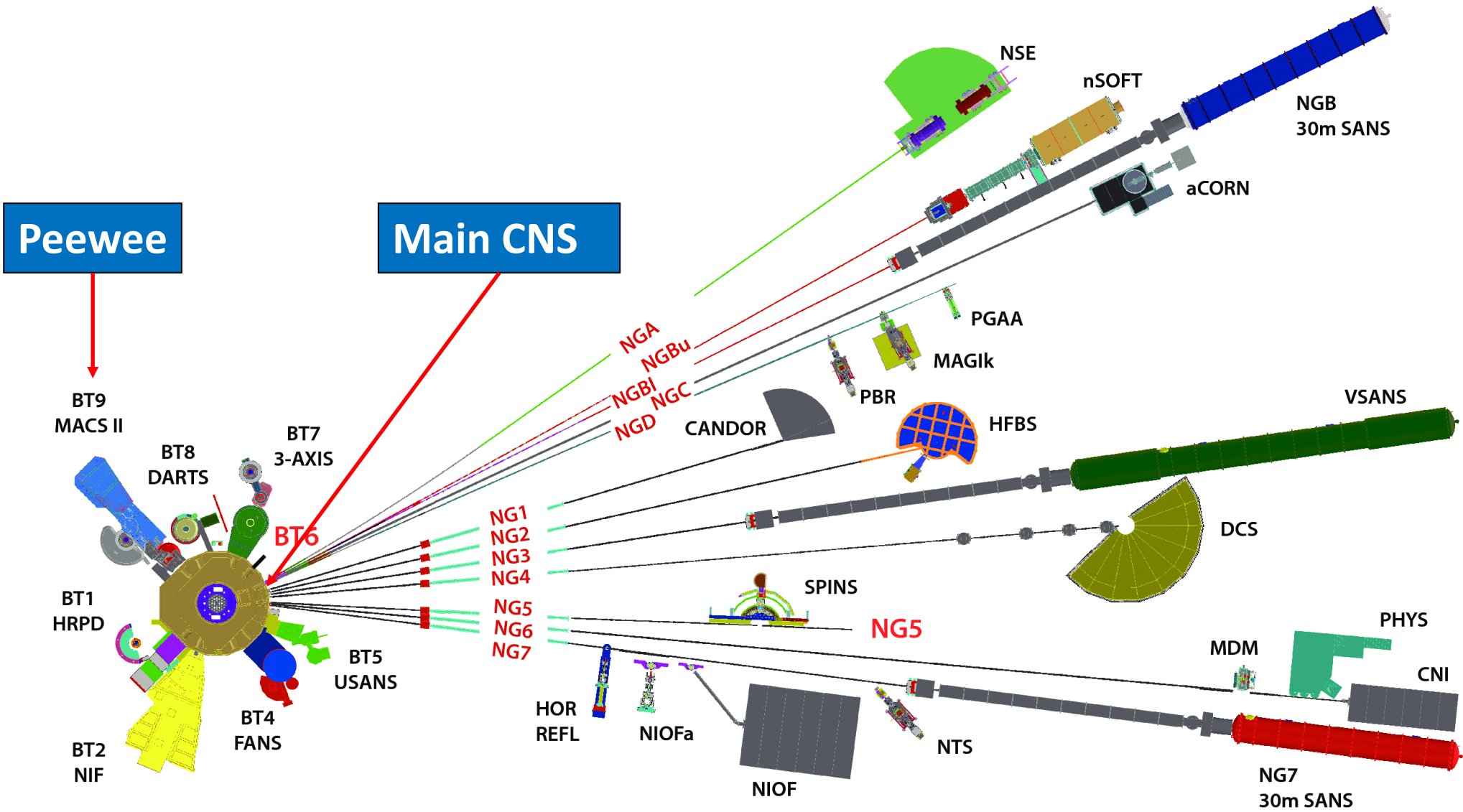


# The NBSR History

- Designed in 1960s (included a beam port for a single cold source)
- It first reached criticality on December 7<sup>th</sup>, 1967
- Initial license was 10 MW
- Upgraded to 20 MW in 1985



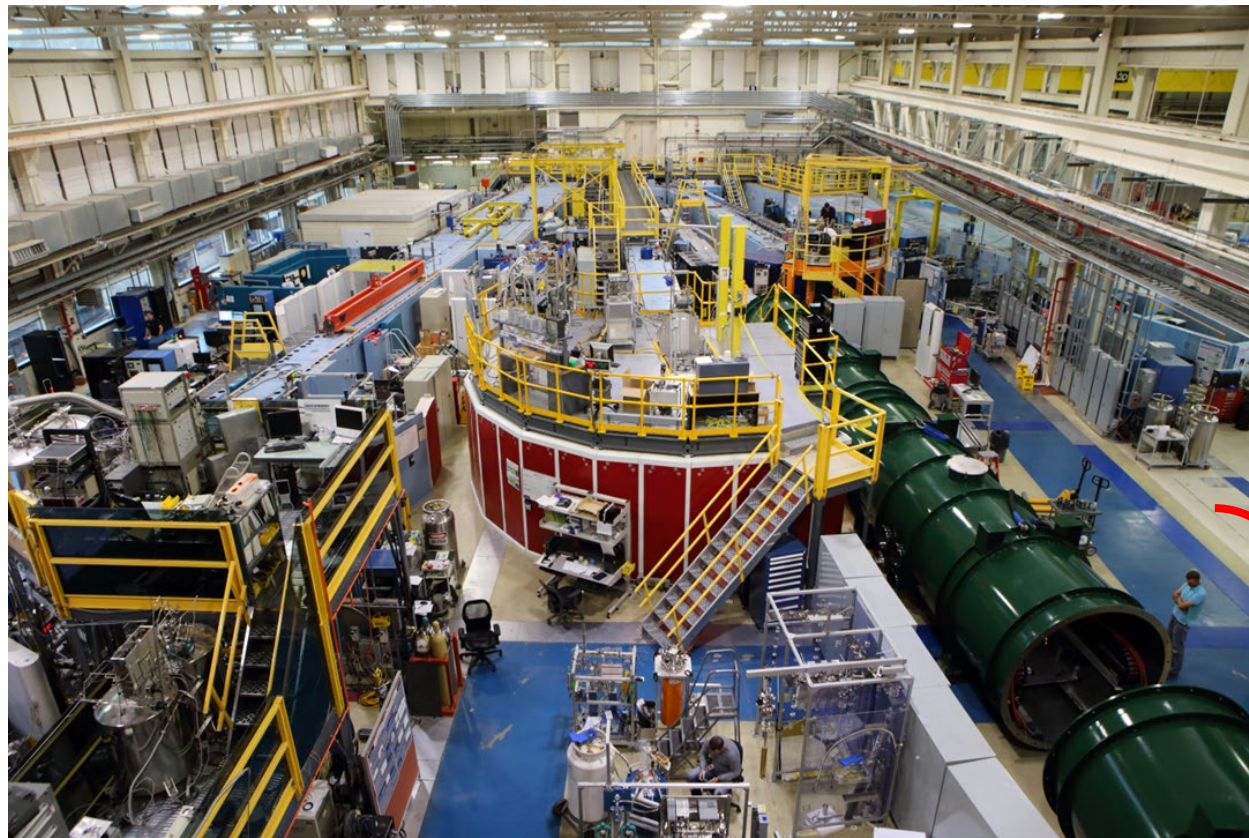
# The NBSR Instruments Layout





# The NBSR Guide Hall

**East**



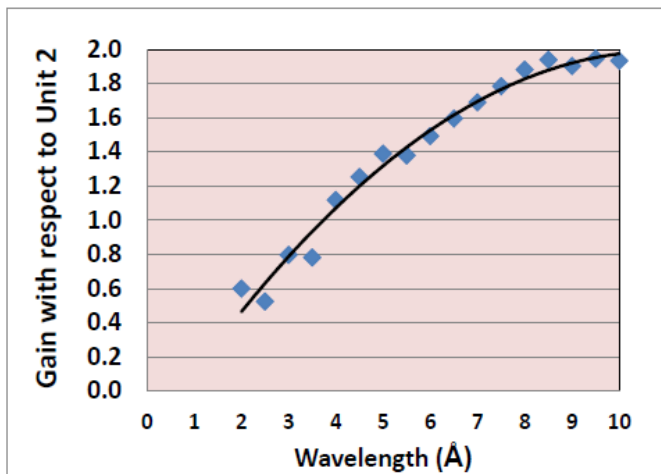
**West**



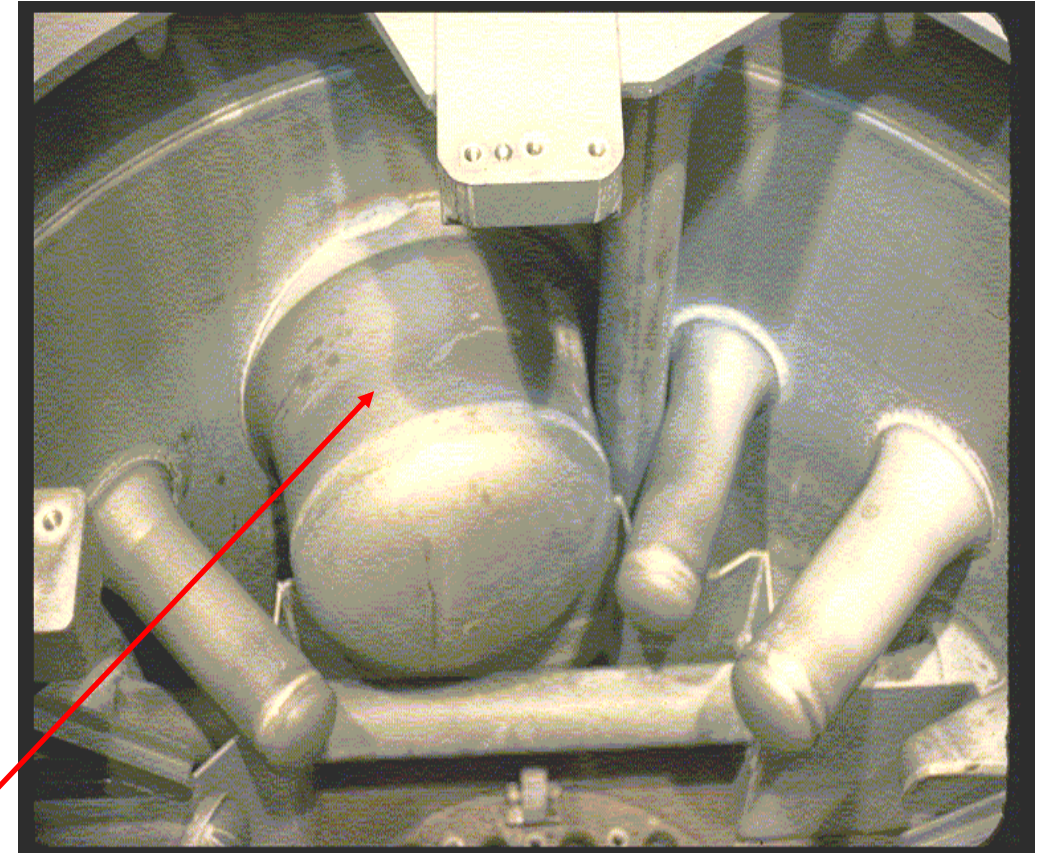


# The Cold Source Through the years

1. D<sub>2</sub>O tank (1967)
2. D<sub>2</sub>O cold neutron source (CNS) installed in 1987  
Gain of 3-5
3. LH<sub>2</sub> CNS installed in 1995  
Gain of ~6x
4. An advanced LH<sub>2</sub> CNS installed in 2002  
Gain of ~2x
5. Peewee (smaller CNS) installed in 2012 in BT-9
6. An upcoming cold source upgrade is planned for 2024



55 cm Diameter



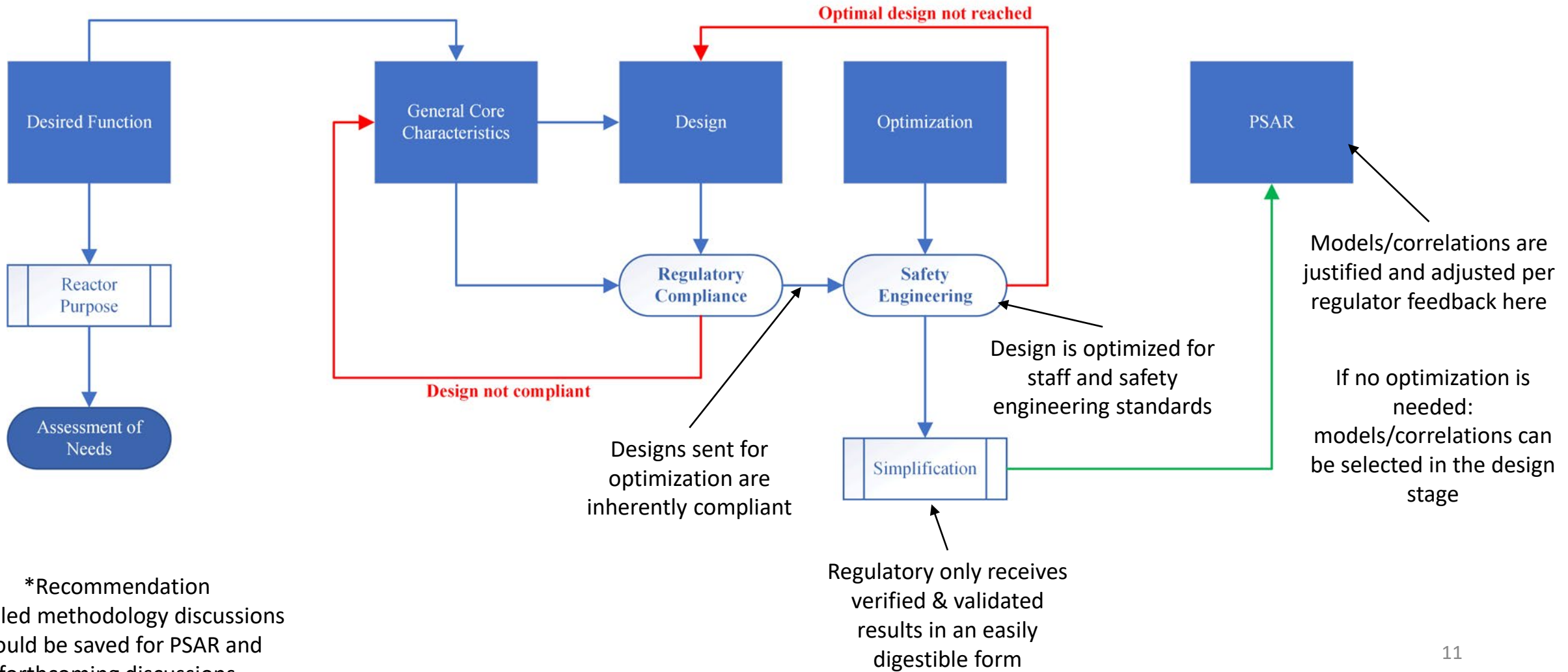


- A study on future options for the NCNR -> replacement reactor is the best option
  - **Refurbish and Replace NBSR Internals**
    - No advancement as a neutron source, and similar cost to building a new reactor.
  - **Keep NBSR Operational**
    - long-term uncertainty in the condition of the reactor
  - **Spallation Source**
    - Steady-state reactor sources are complementary to pulsed accelerator-based neutron sources
  - **New Core inside NBSR**
    - Retains the ageing issues of NBSR, and it is likely as expensive as a new reactor.
  - **New Reactor**
    - A clean slate: ultimately the best path forward. Allows “Ideal” neutron source design

- Proposed design approach with emphasis on modeling development and acceptance processes
- Physics decomposition is suggested to reduce costs and efforts for validations
- Emphasis is placed on regulatory compliance and pre-engagement activities
- Suggested optimization process encourages sensitivity coefficients/behavior computations as a function of varying design parameters
- Factors of safety recommendations are given for both input uncertainties and sensitivity metrics.

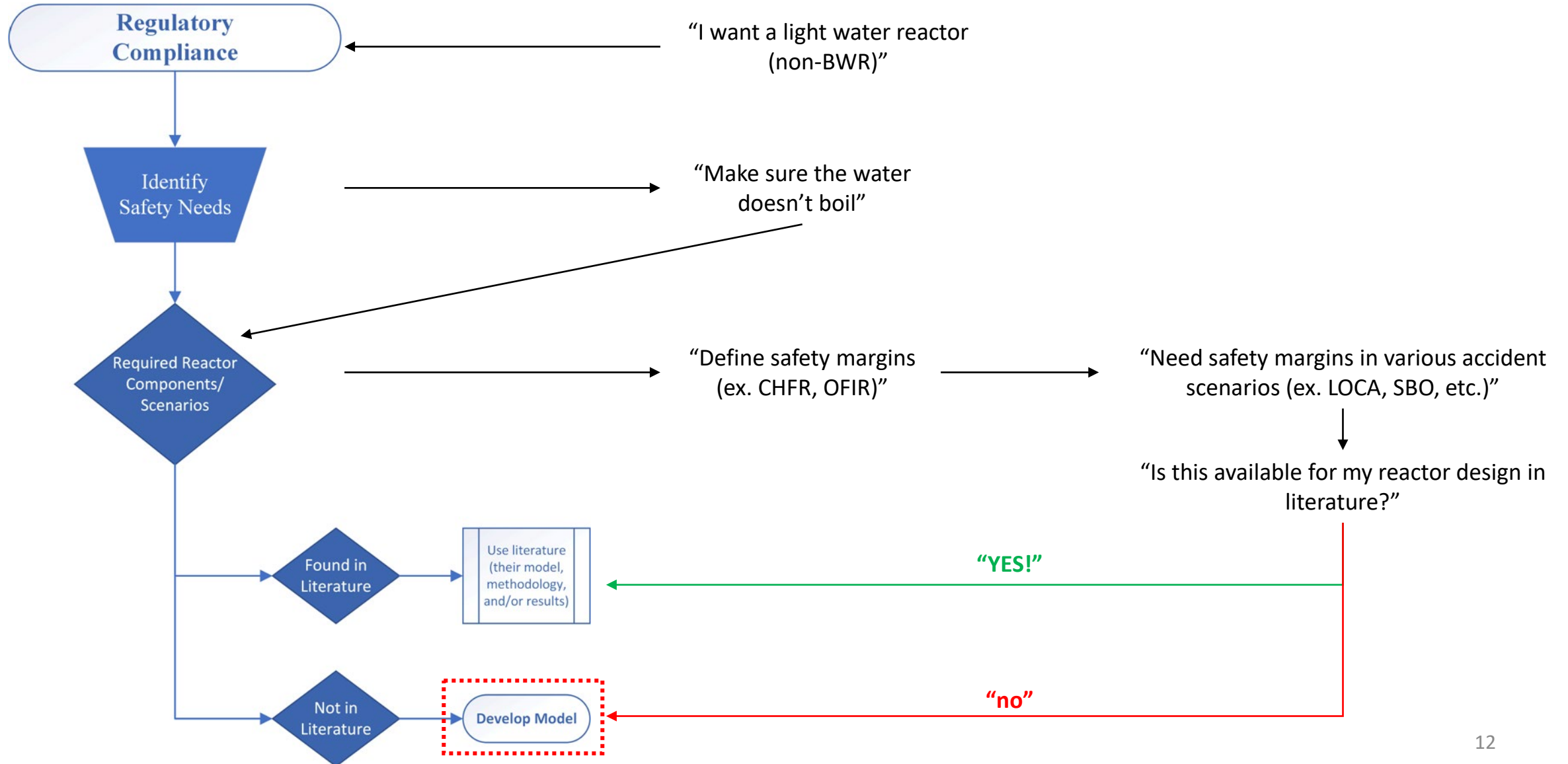


# Proposed Design Approach



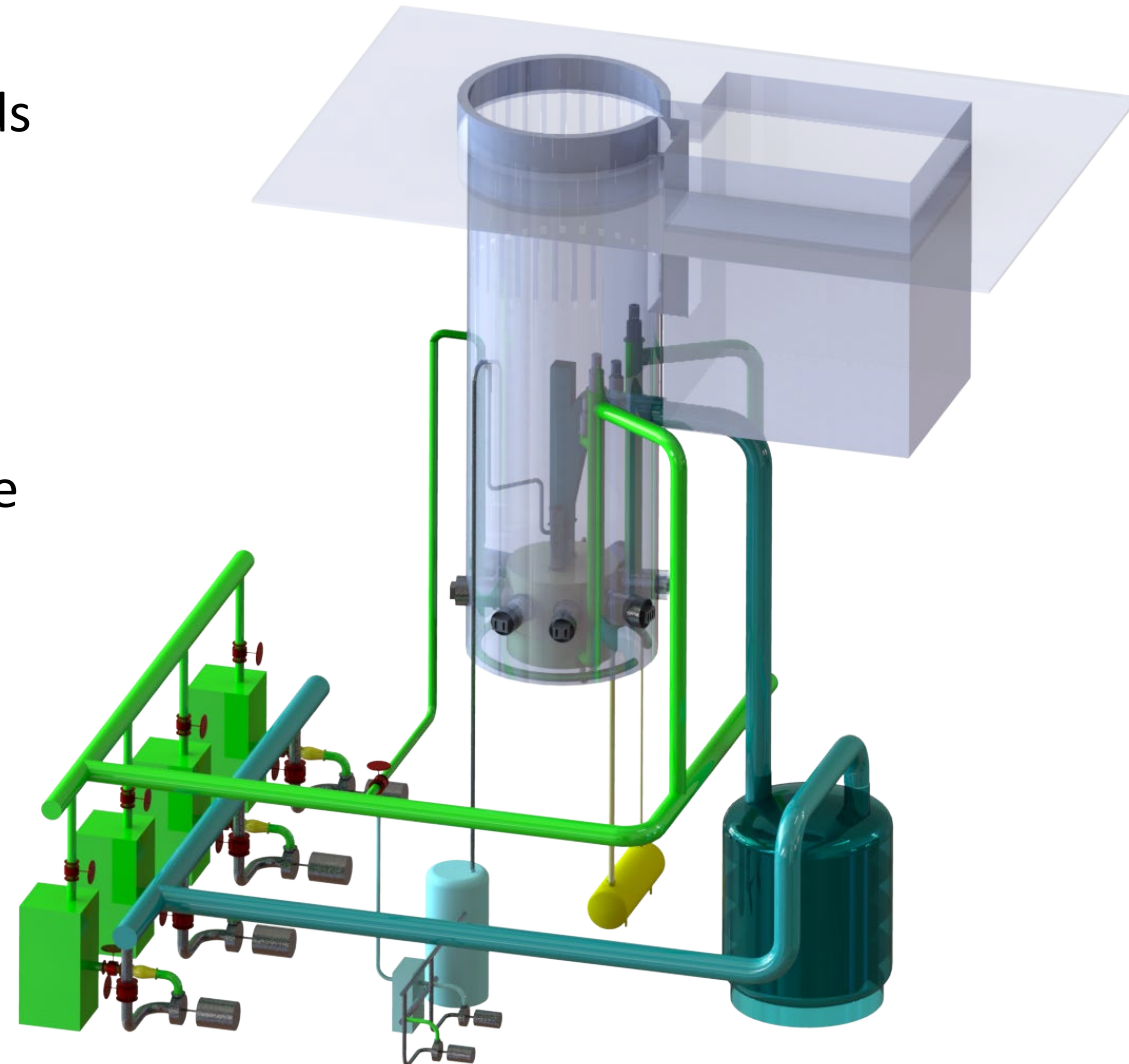
\*Recommendation  
Detailed methodology discussions  
should be saved for PSAR and  
forthcoming discussions

# Regulatory Compliance



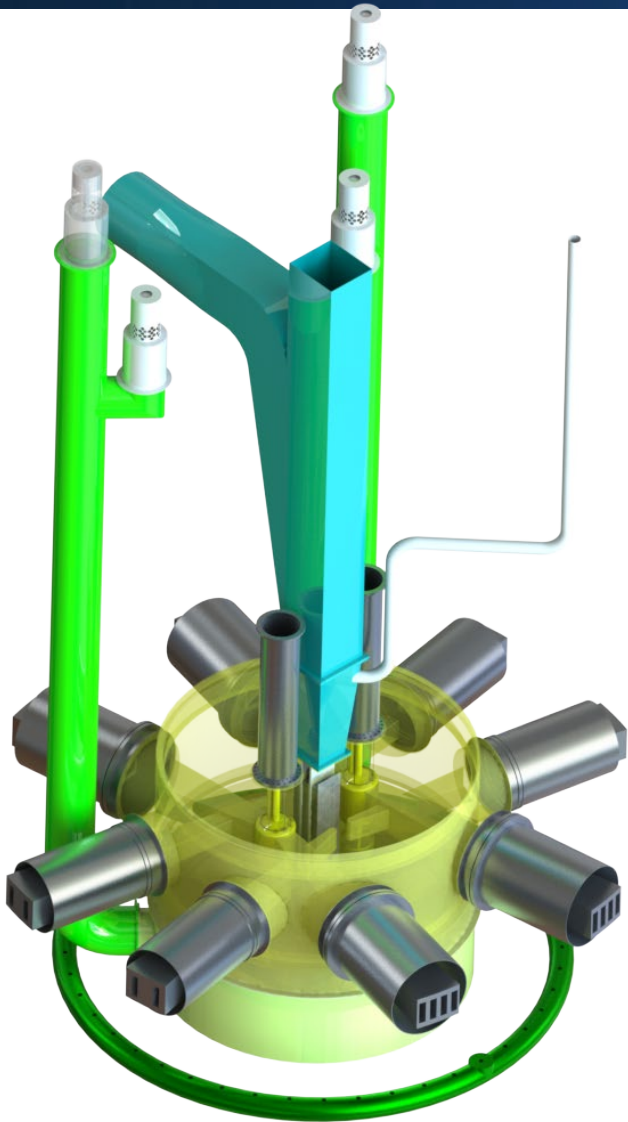
# Design of NNS

- Function is to provide cold and thermal neutron beams that meet the current and anticipated needs of the neutron science community with high availability
- Based on proven technologies
- Ignore concept level details, mechanical and structural components
- Utilize operational experience on High Performance Research Reactors in the US and abroad
- Optimize the reactor for neutron research
- Establish design specifications and requirements based on a manufacturable design
- Pre-engage with regulator to ensure design specifications meets licensing requirements



Reactor Pool and Primary Coolant System

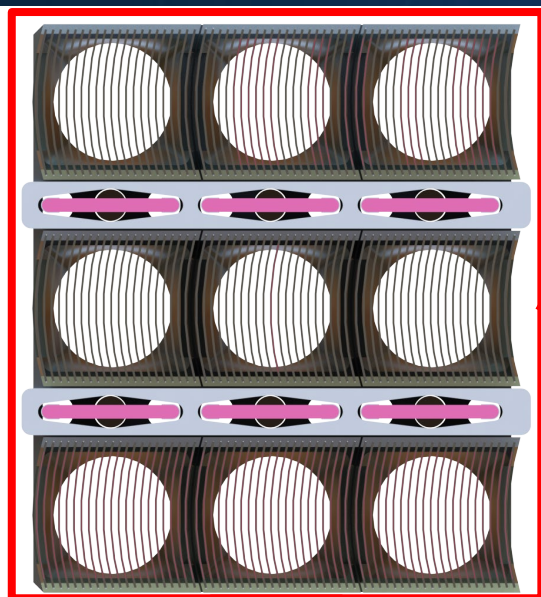




- Maximize thermal neutron density outside the core
- Avoid unproven, untested, or any new technology unless for substantial safety gain
- Influenced by several reactors designed for neutron science
- Nominal power of 20 MW
- U-10Mo LEU (or  $U_3Si_2$ , U-ZrH etc.), utilize “qualified” fuel
- Light-water-cooled compact reactor core
- Surrounded by heavy-water reflector tank
- 2 Cold Neutron Sources
- 8 Thermal Neutron Beams
- 40 days operating cycle

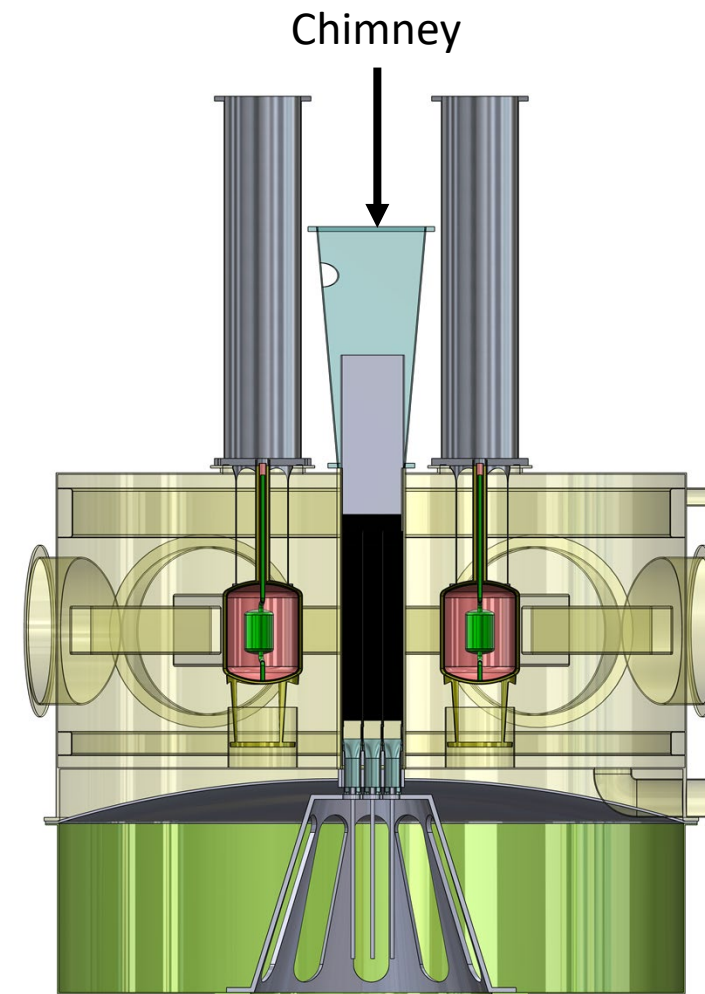
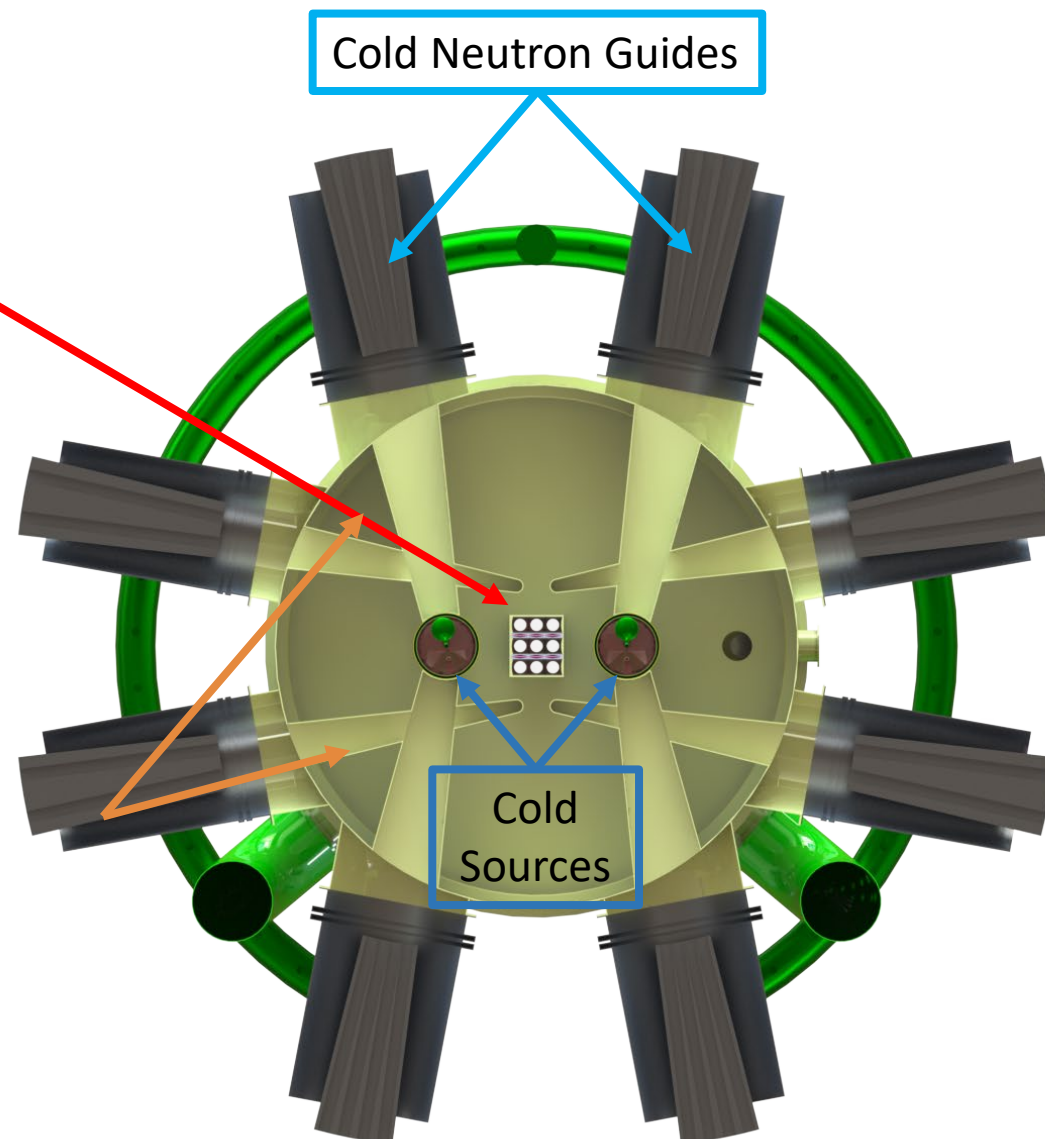
**Reflector tank with surrounding features**

# Design of NNS



Core

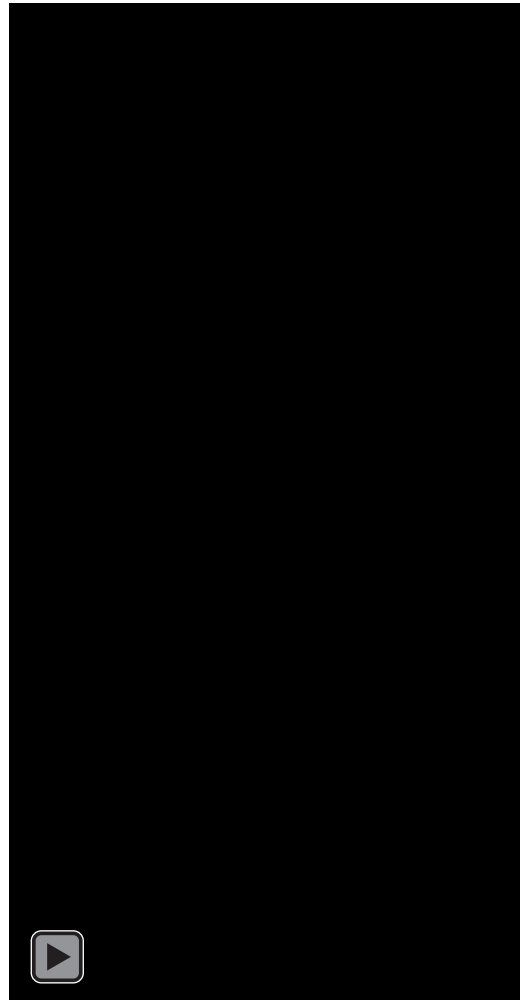
Thermal Beams



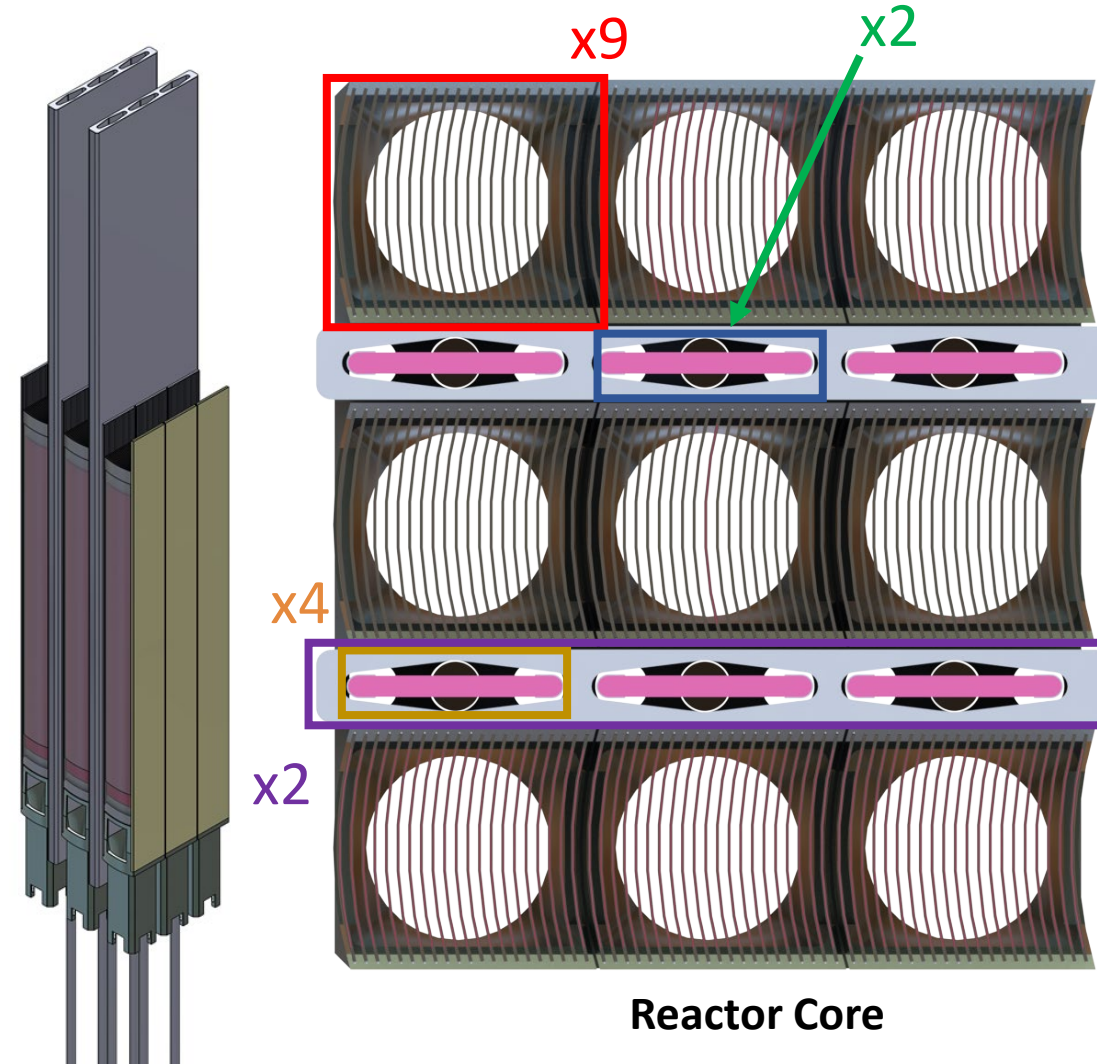
Reflector Tank with the core, cold sources, and beam tubes

# Design of NNS

- Nine **fuel assemblies** (FA) in a 3x3 array
- Each FA contains 21 U-10Mo fuel plates
- 19.75% enriched Y-12 fuel wrapped with  $\sim 8 \mu\text{m}$  thick zirconium foil
- Four **control blades** and two **safety blades** placed in the center within two **guide boxes**
- Core horizontally divided into three rows
- 64 coolant channels at each row
- Optimize fuel cycle length & maintain a negative reactivity feedback



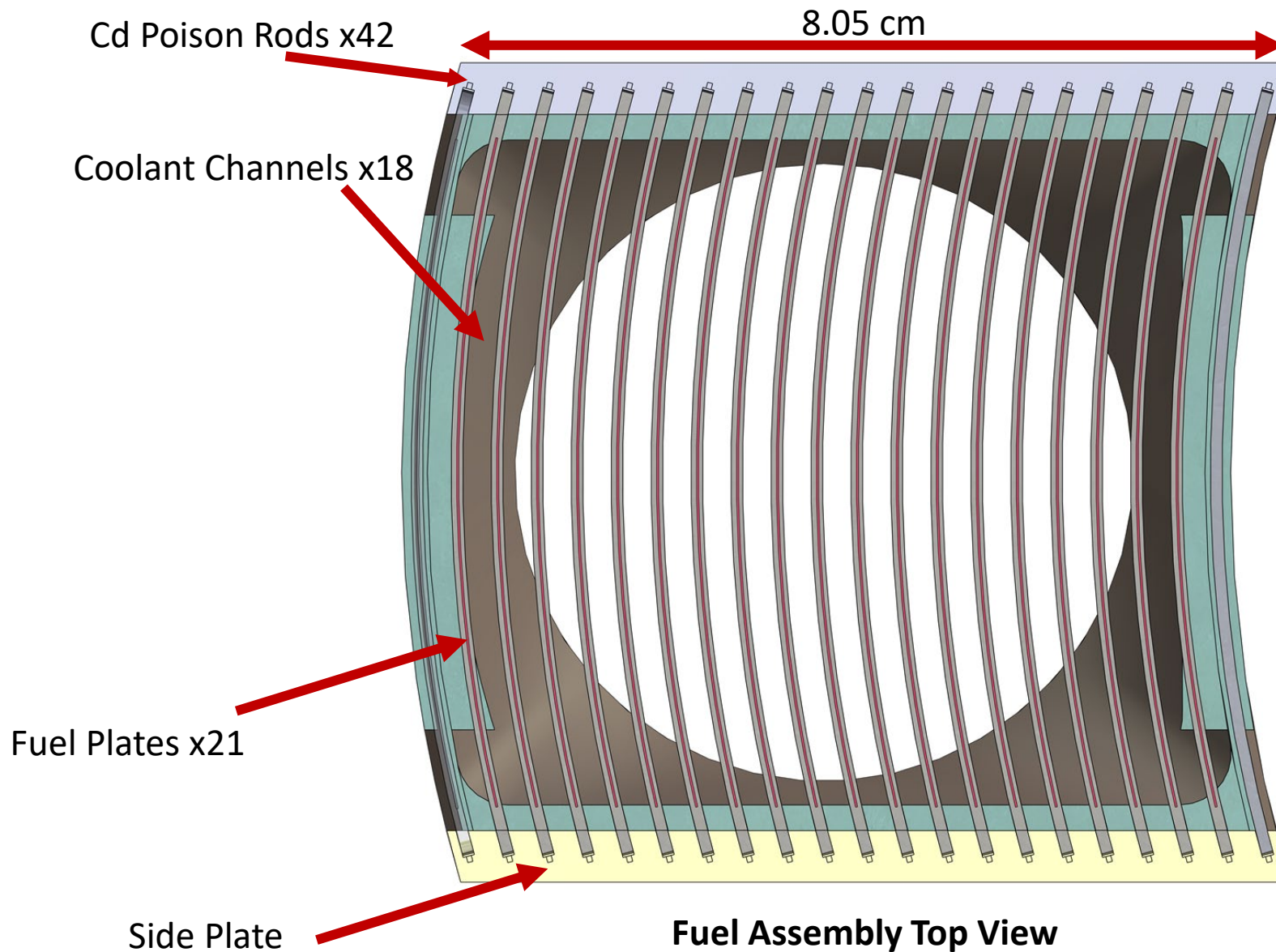
Fuel Assembly



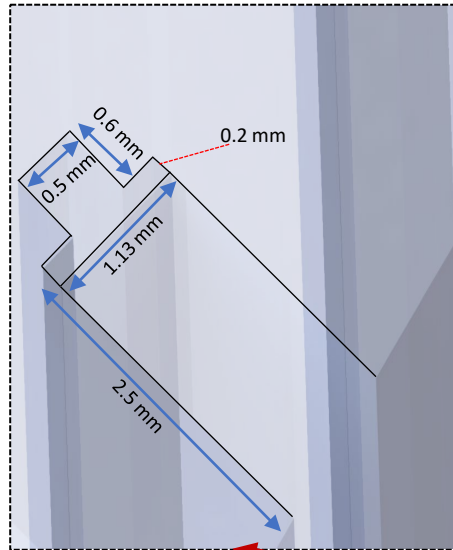
Reactor Core



# Design of NNS

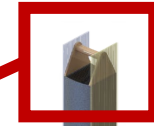
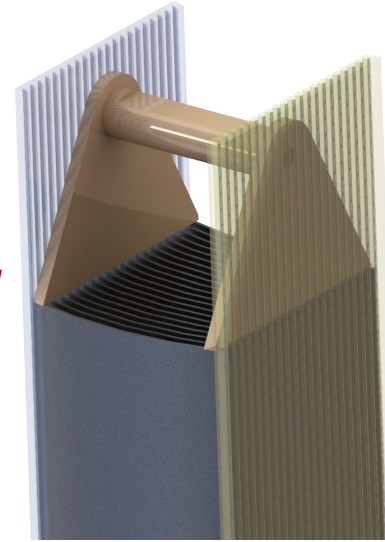
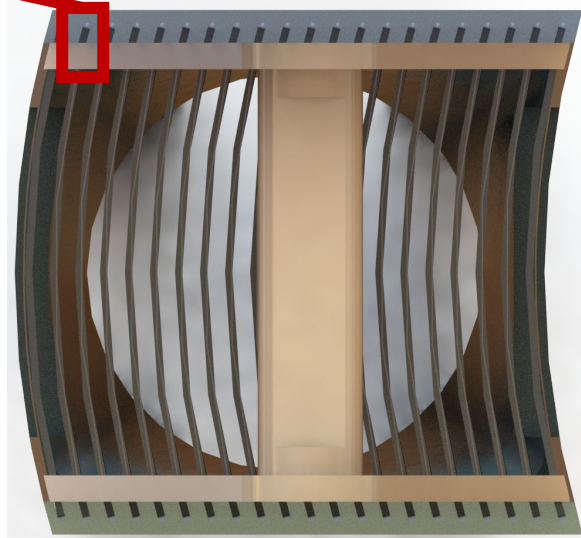


# Fuel Assembly Details

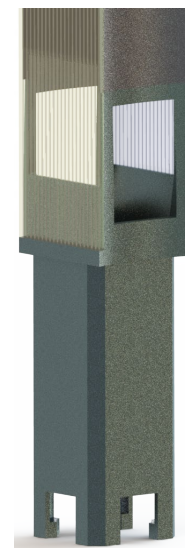
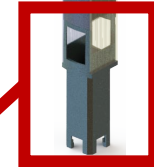


Fuel Plate

Top View



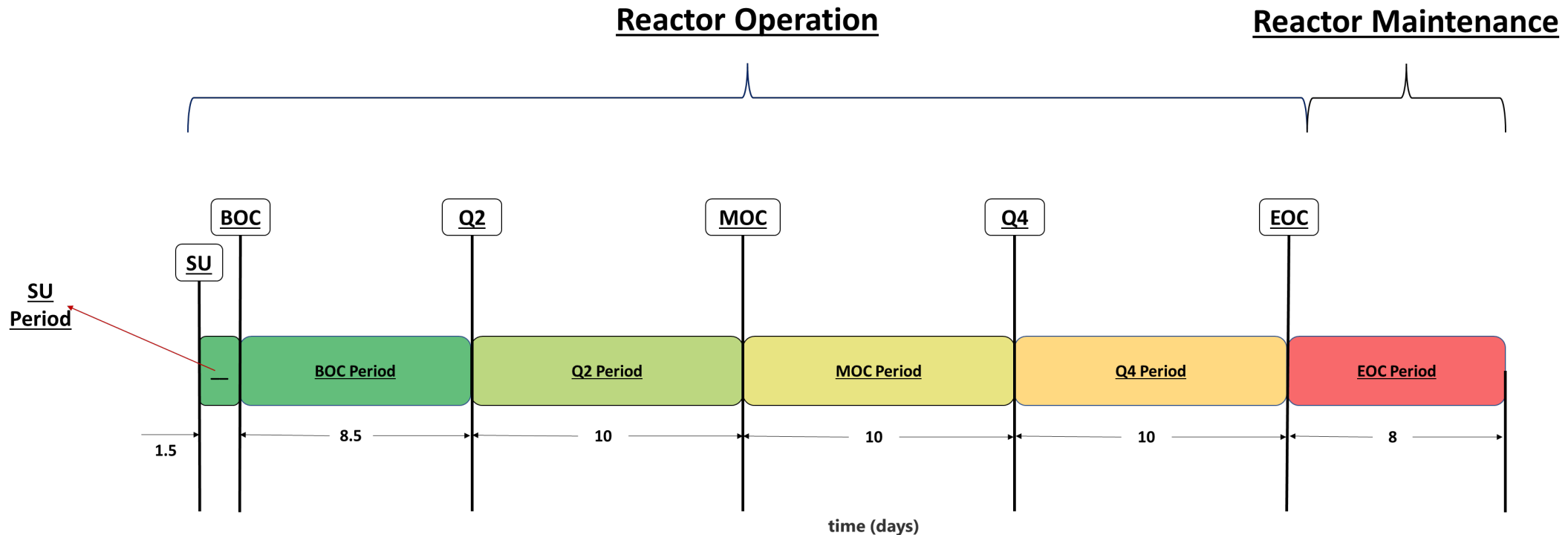
Fuel Assembly



- The number of FAs for any core loading is 9
- The reactor has two independent and diverse shutdown systems.
  - Safety Blades
  - Reflector Dump system
- Designed as a high leakage core with a compact structure
- Neutronics analysis is performed via the Monte Carlo N-particle Code (MCNP) & ENDF/B VIII.0
- The thermal treatment of the materials has not been used
- Burnable poison Cd-rods in fuel assemblies not optimized
- Core size, width/height not fully optimized for maximum cold source brightness
- Cold source size/locations not fully optimized for maximum brightness



# Fuel Cycle States Analyzed



**SU** is the startup cycle state starts with the 0th day of a cycle. In this state, either the initial core loading compositions

**BOC** cycle state covers the first one-quarter of a 40-day cycle of operation.

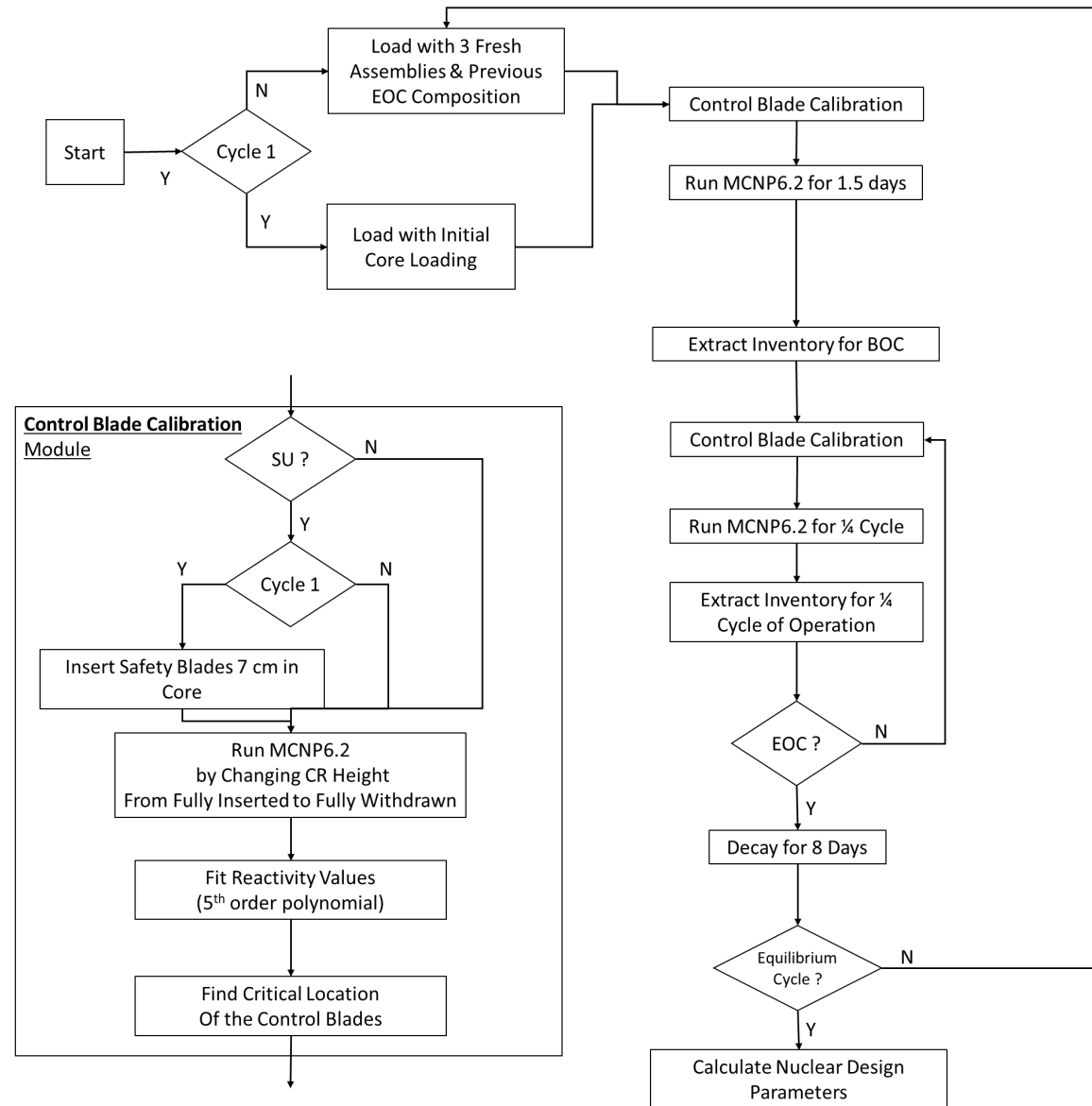
**Q2** cycle state is designed to eliminate possible errors that can arise from the constant location of the control blades while moving from the BOC to the MOC cycle state.

**MOC** cycle state covers the third quarter of a 40-days cycle operation starting from the exact middle of the operating cycle.

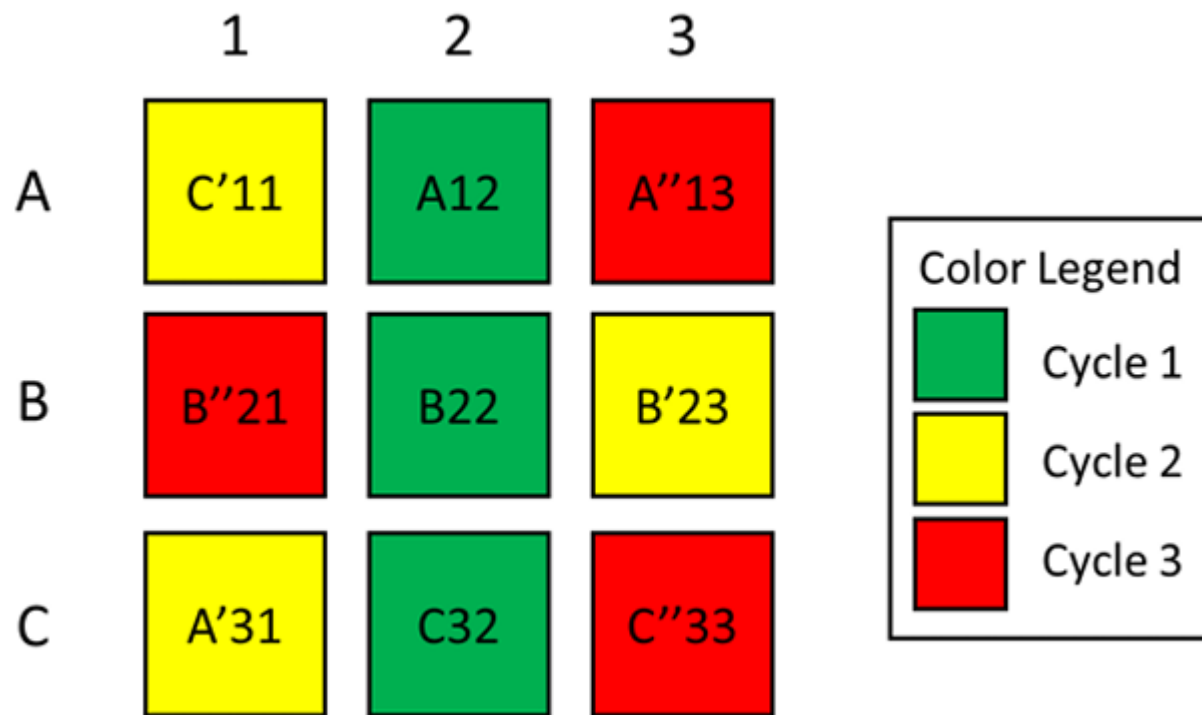
**Q4** cycle state covers the last quarter of a 40-day cycle of operation.

**EOC** cycle state is the final part of a cycle that covers 8 days of decaying of short-lived isotopes in the maintenance period of the reactor prior to the next operational cycle.

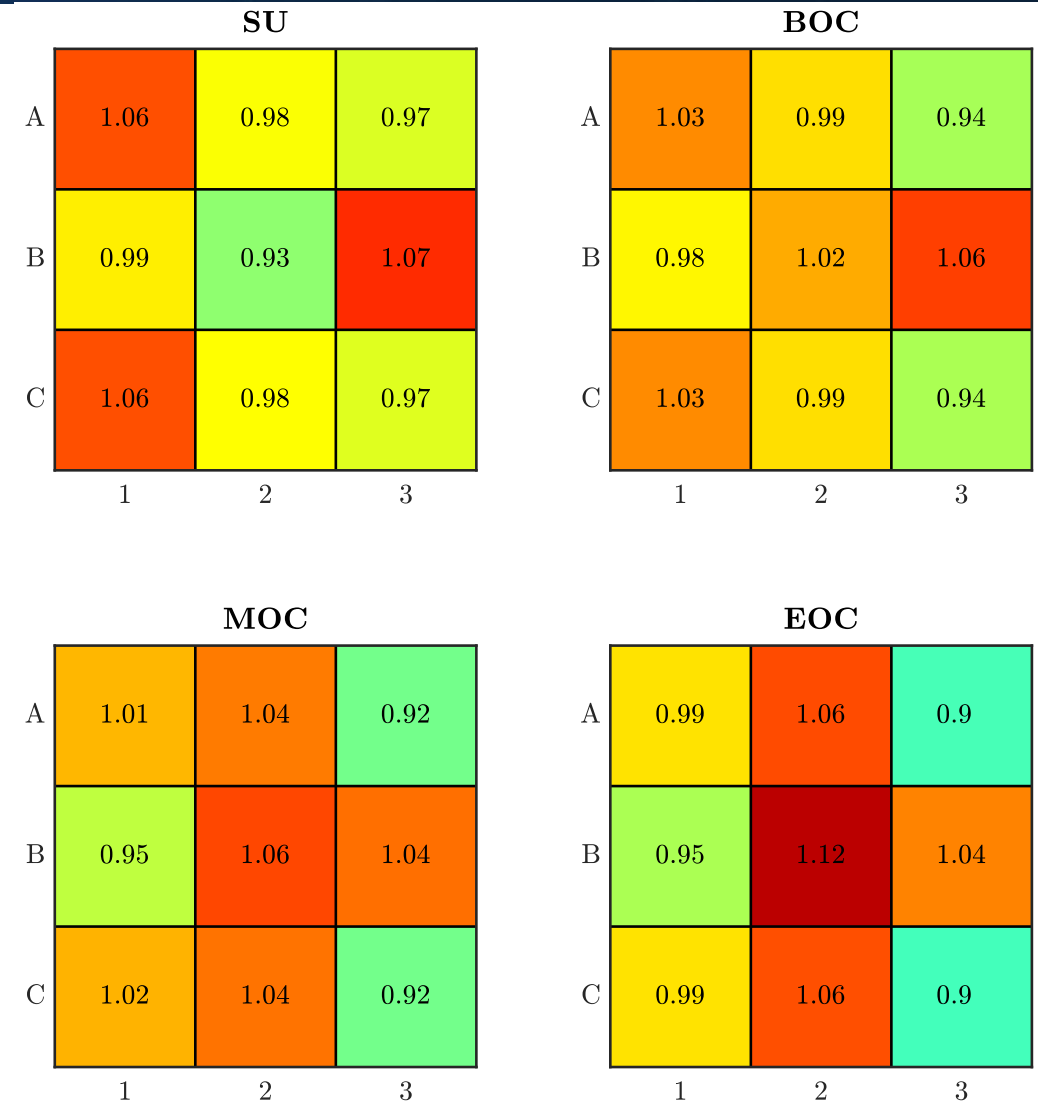
# Equilibrium Core



# Core Power Analyses



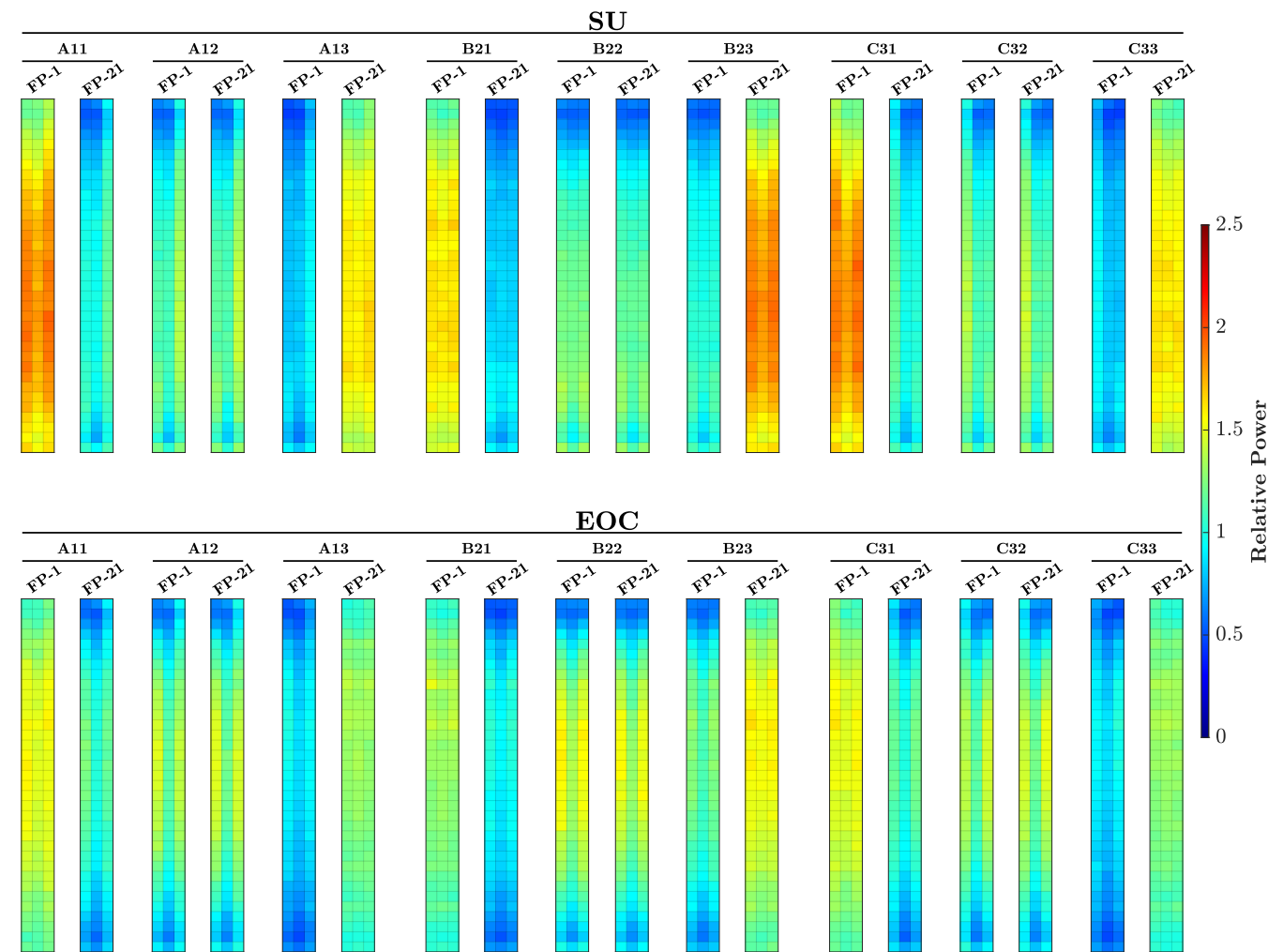
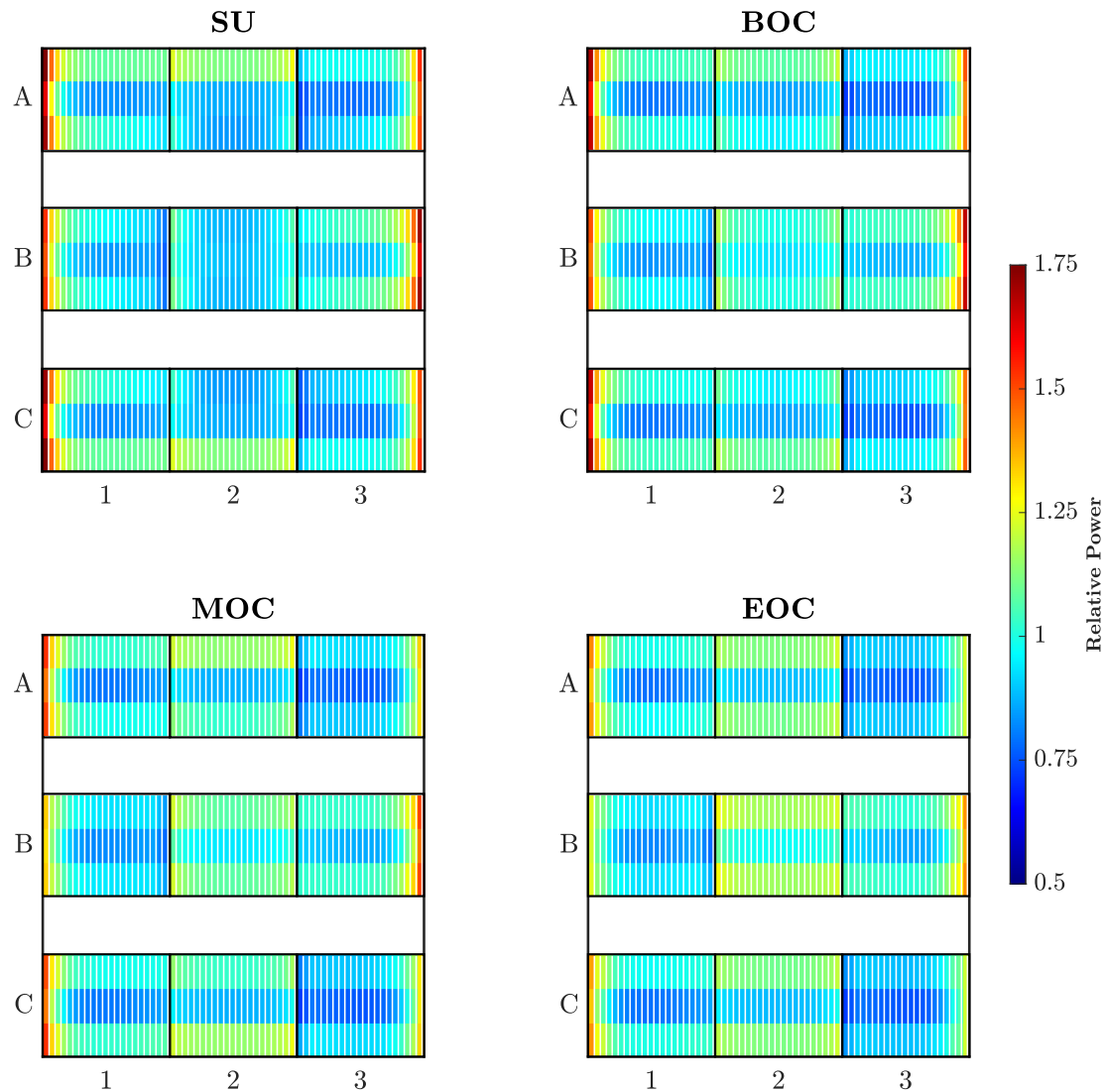
**NNS Current Fuel Management Scheme**



**Normalized Power Heatmap of Assemblies in Each Cycle State**



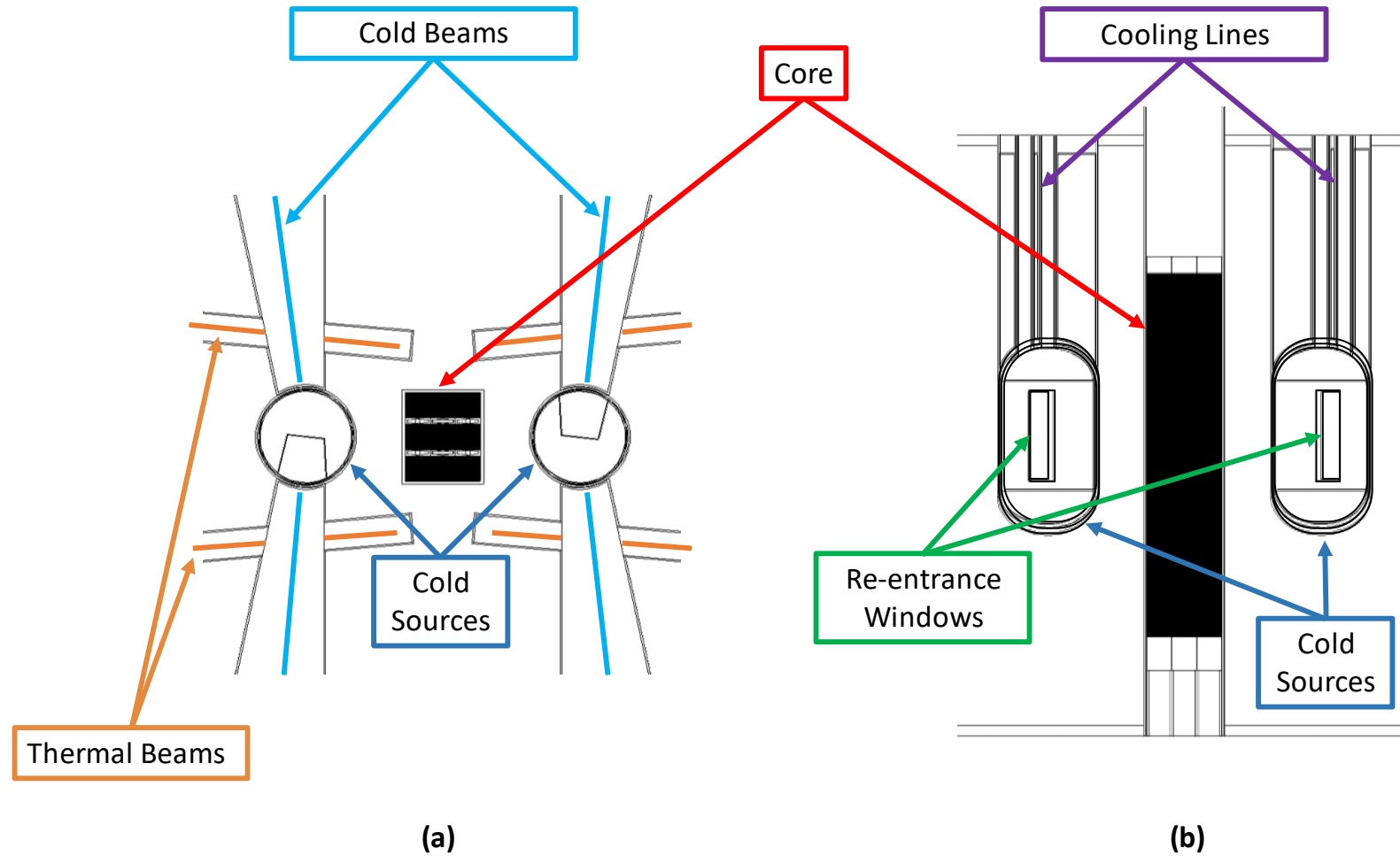
# Core Power Analyses



Axial-averaged relative powers of each fuel plate

Detailed relative power distributions of the 1<sup>st</sup> and last fuel plate in each assembly

# Core Neutron Density Distributions



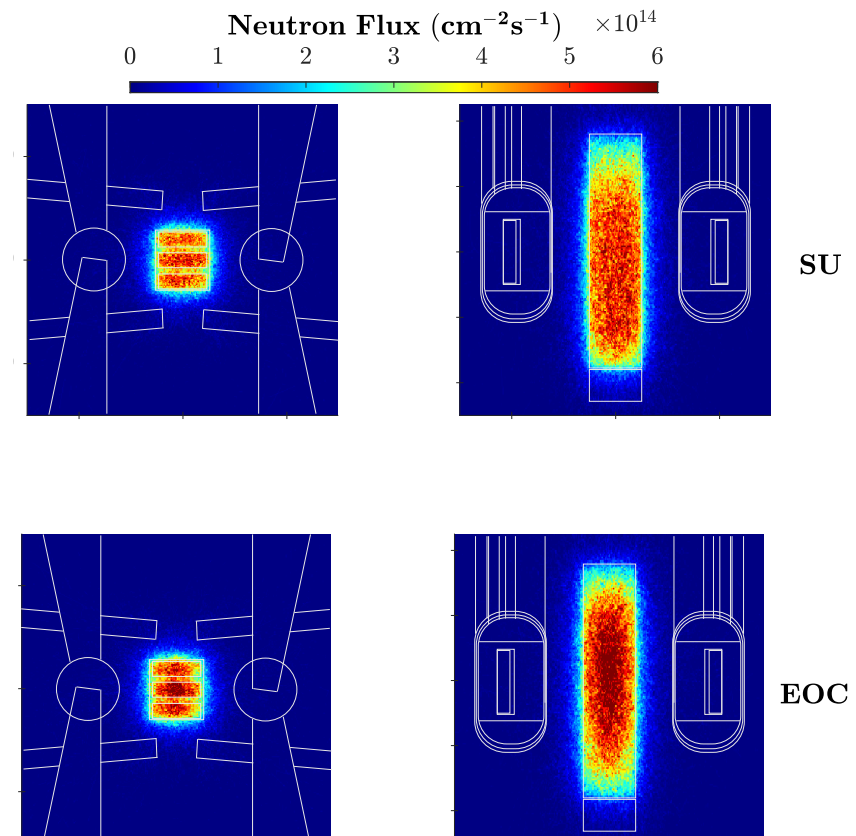
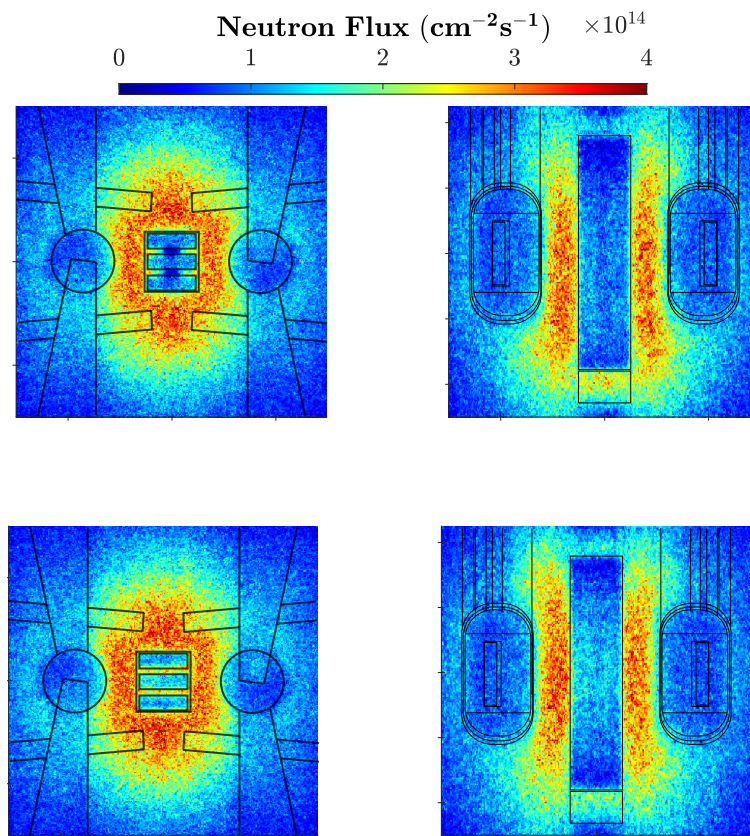
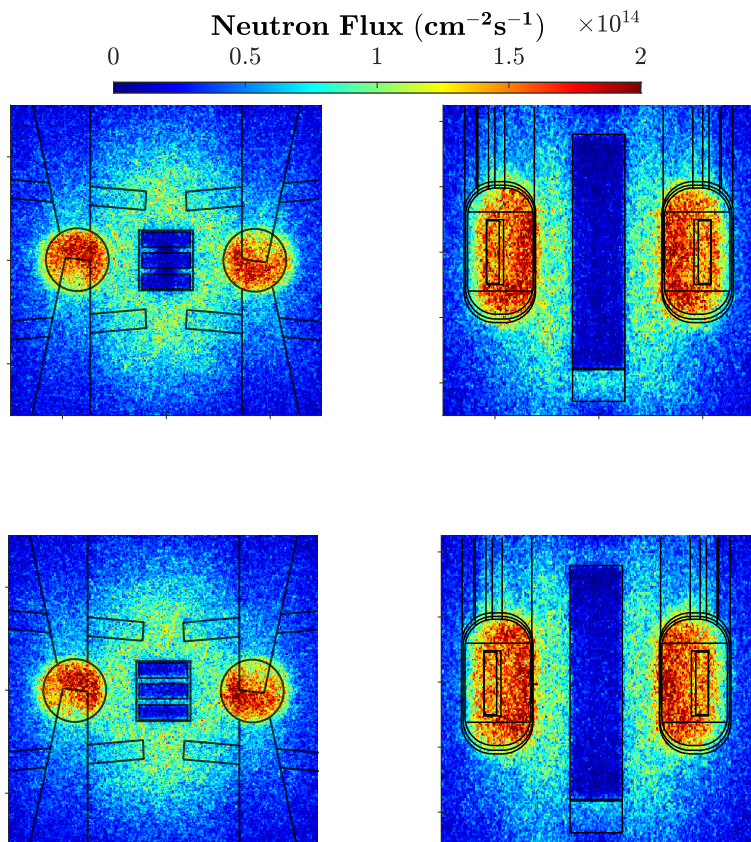
Representation of the core, CNSs, and their peripheral components in (a) the planar view and in (b) the elevation view

# Core Neutron Flux Distributions

**“Cold”**  
**(<0.025 eV)**

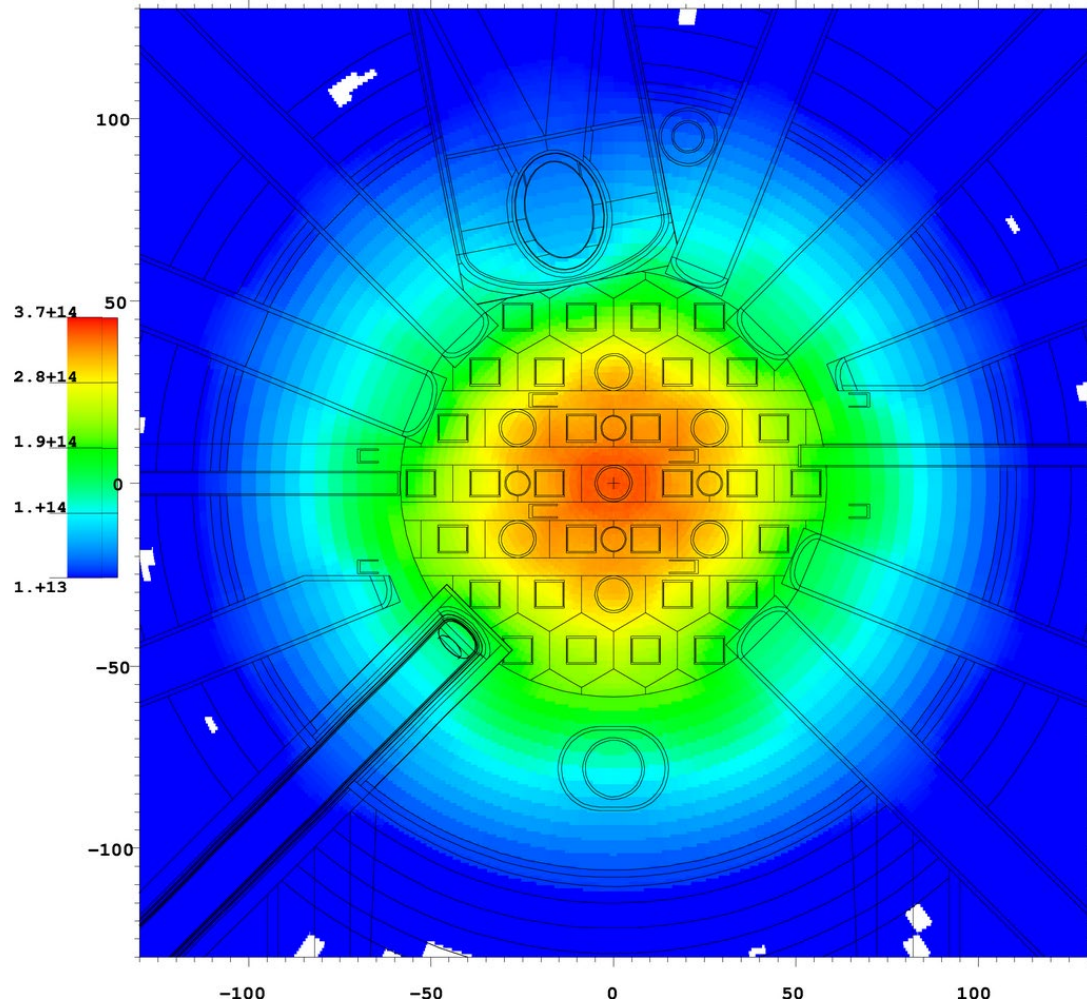
**“Thermal”**  
**(0.025 – 0.4 eV)**

**Fast**  
**(100 keV – 20 MeV)**

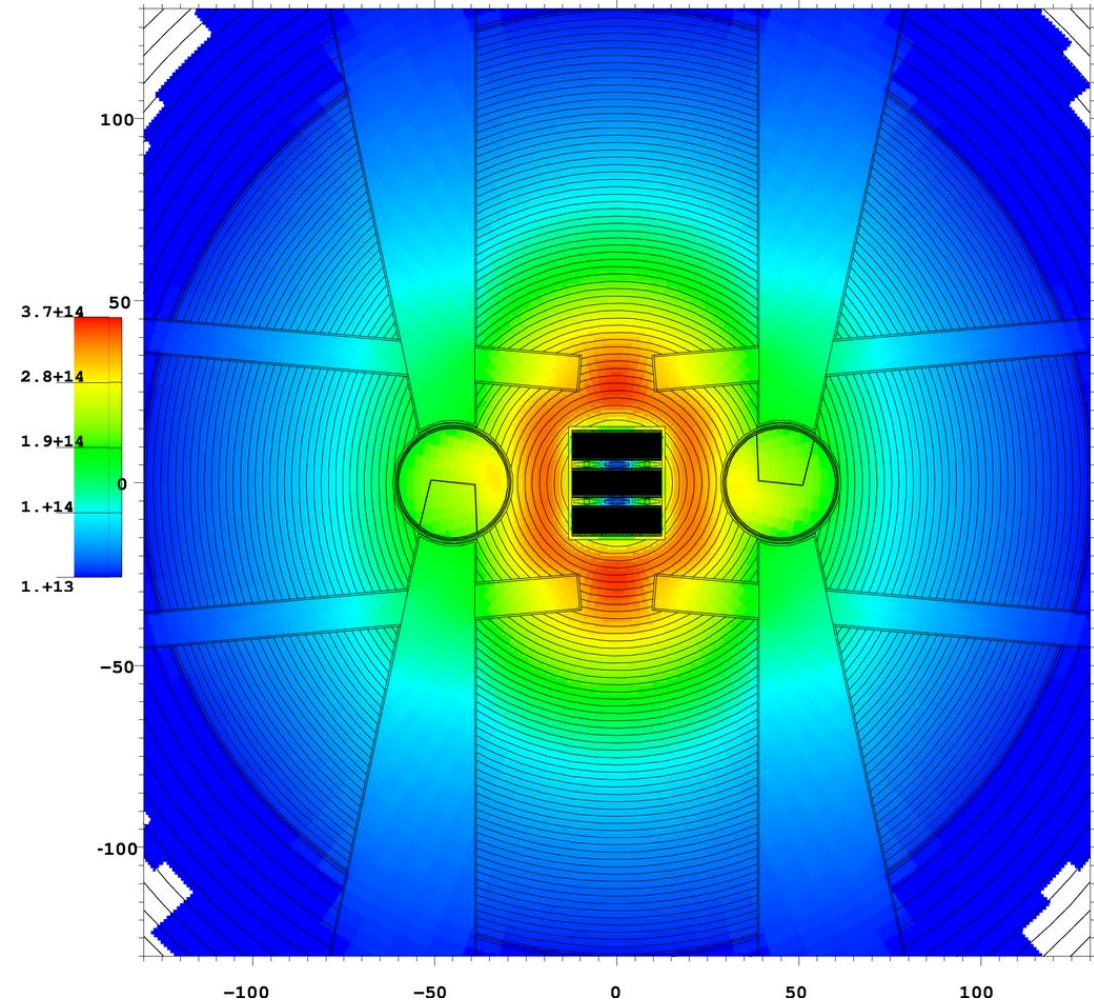




# Thermal (<0.3eV) neutron flux



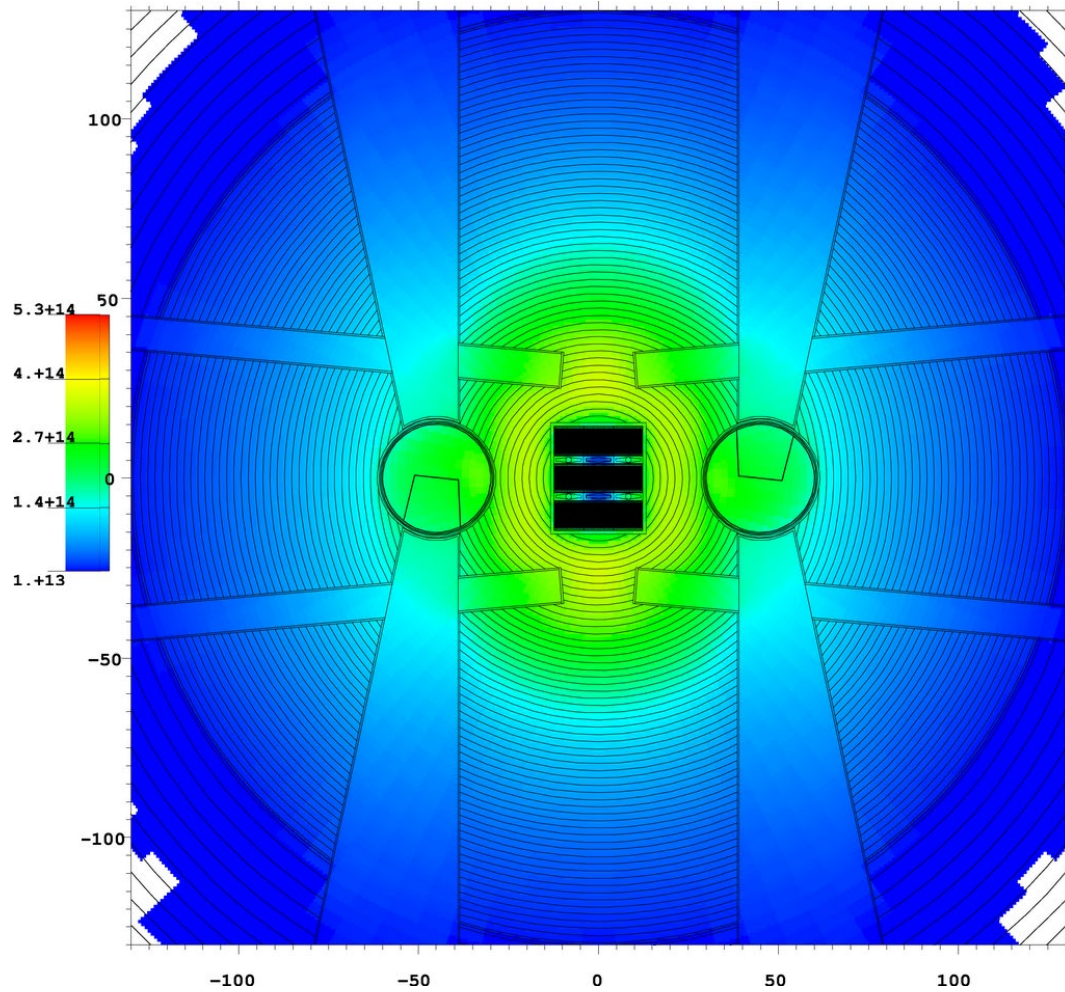
**NBSR**



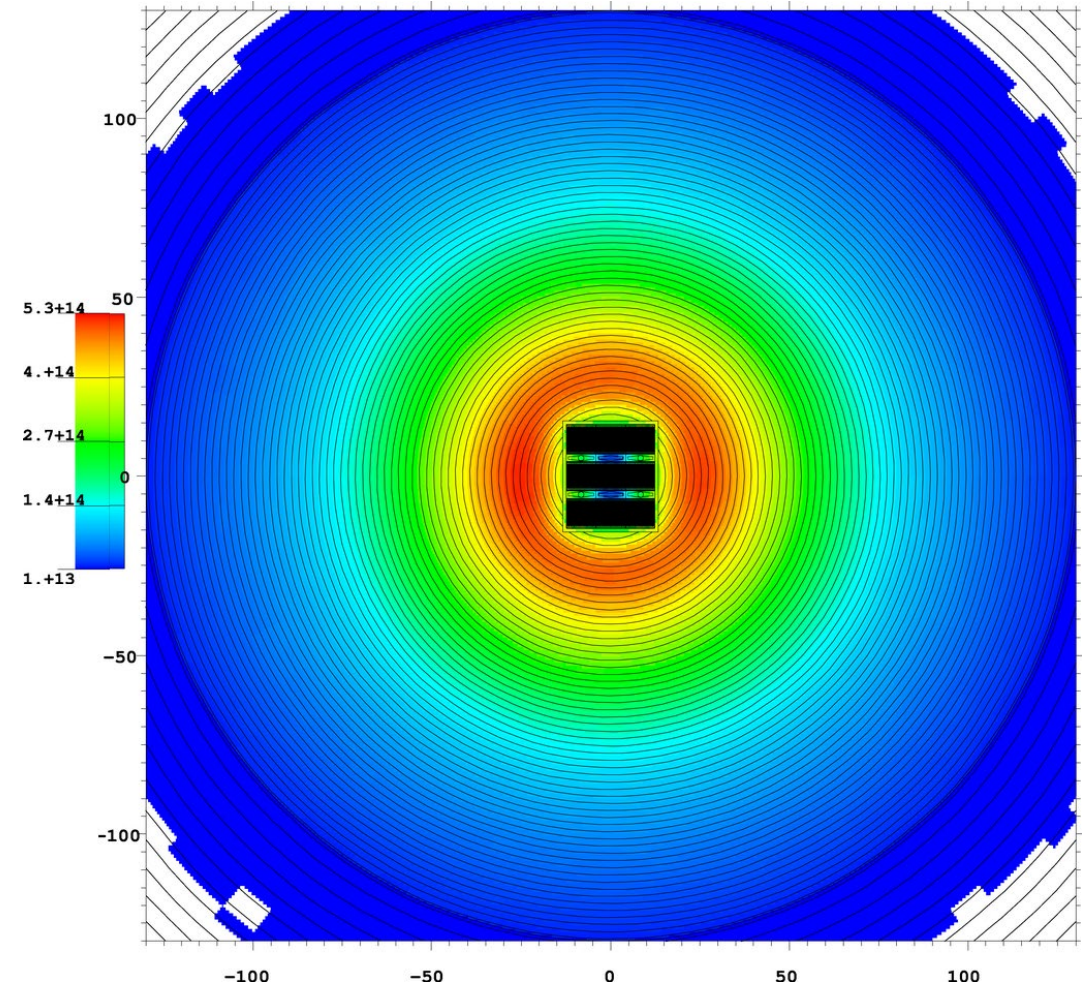
**NNS**



# Perturbed & Unperturbed Neutron Flux



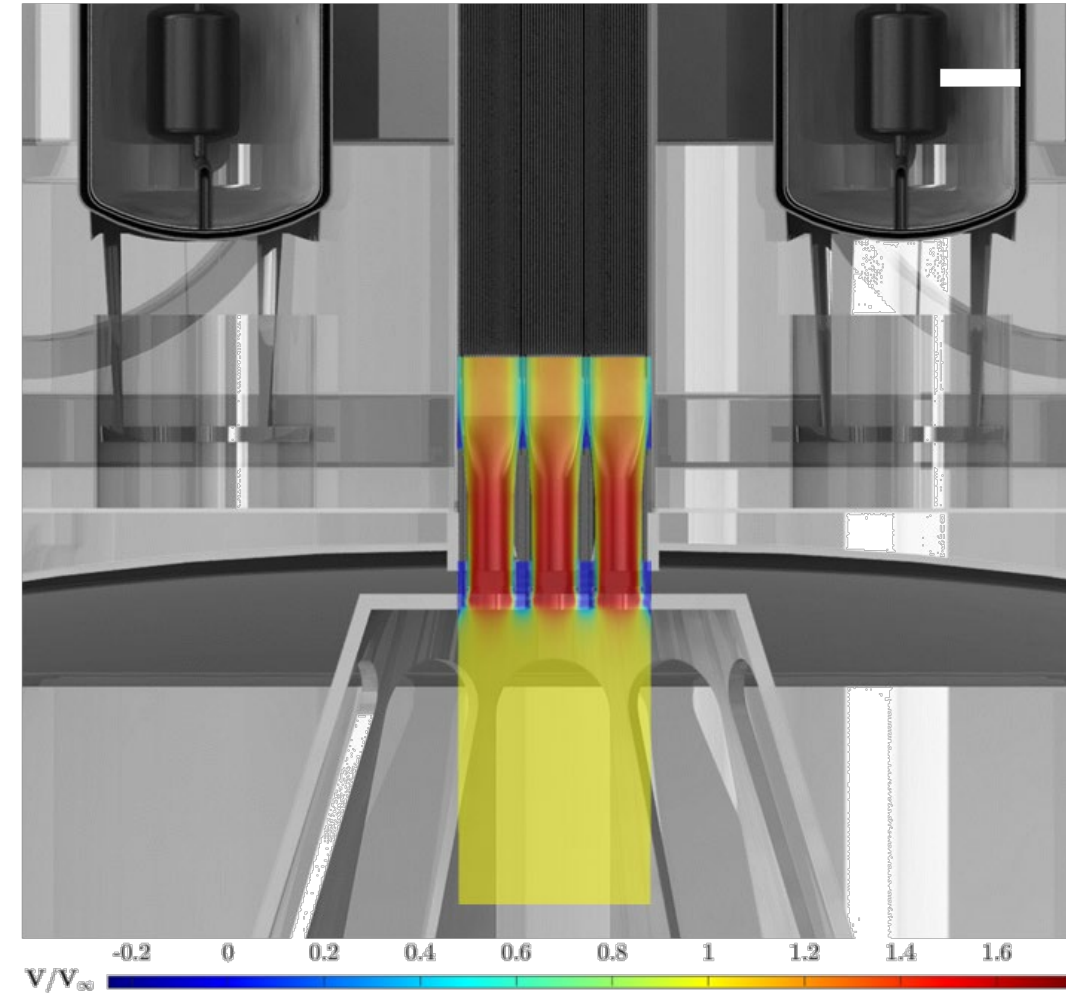
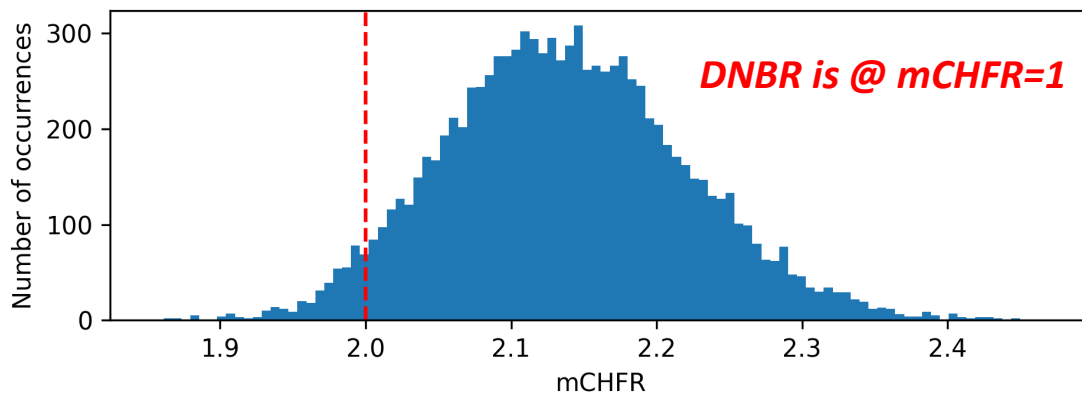
**NNS  
(Perturbed)**



**NNS  
(Unperturbed)**

# Core Thermal Hydraulics Safety

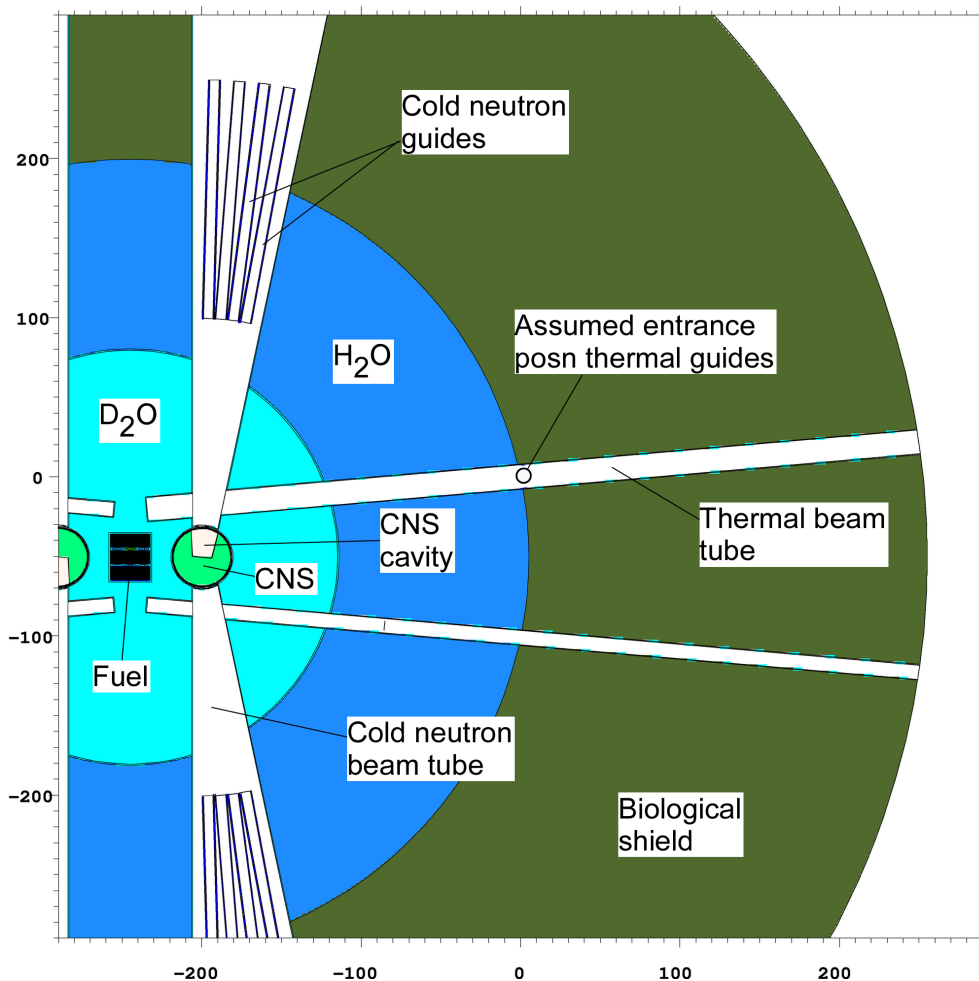
- Analyses show no departure from nucleate boiling (DNBR) at any cycle state.
- Maximum bulk temperature was nearly uniform in all channels and remains below 335 K.
- Maximum cladding temperature remains below 365K.
- CFD work is ongoing to assess the flow behavior at the inlet of the core (do we have undercooled channels?).
- Work is starting to assess the design of the chimney and the outlet of the NNS (how tall should it be? how should the outlet connection look?).



CFD Model of Normalized Streamwise Velocity Profile of NNS Inlet



# Proposed Cold Neutron Instruments



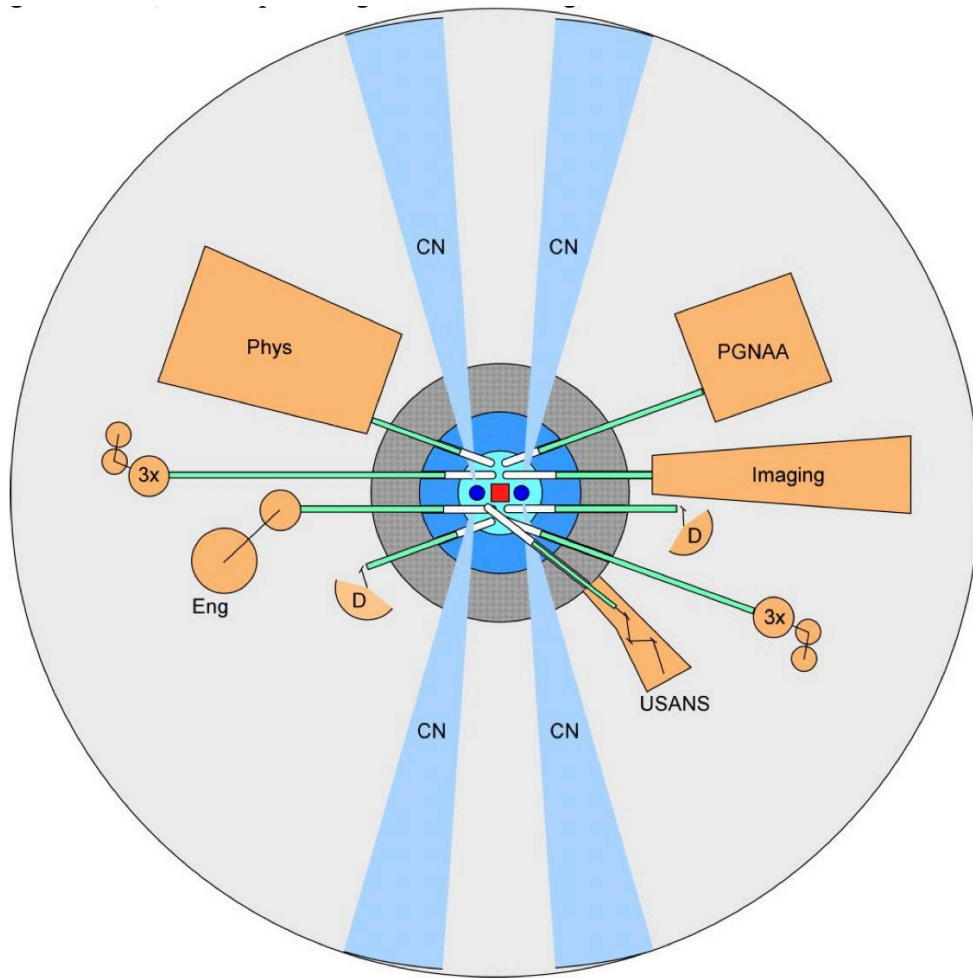
Instrument type	Total Number	End position
Small-Angle Neutron Scattering (SANS)	2-3	YES
Reflectometer (CANDOR type)	2	YES
Cold Neutron Imaging (CNI)	2	YES
Cold 3-Axis (CN3X)	2	YES
Backscattering (BS)	2	YES/NO?
Neutron Spin-Echo (NSE) (Mezei-type)	1	YES
Neutron Spin-Echo (NSE) (WASP type)	1	YES
High current physics experimental position (Physics)	1	YES
Prompt Gamma Activation Analysis (PGAA)	1	YES
Neutron Depth Profiling (NDP)	1	YES
Materials Diffractometer ( $\lambda > 0.3$ nm)?	1?	YES
Interferometer	1?	NO
Monochromatic Physical Measurements Laboratory (PML) positions	2-3?	NO
Miscellaneous monochromatic/ test positions	2-3?	NO
Very Small-Angle Neutron Scattering (vSANS)	1	YES
<b>TOTAL</b>	<b>22-25</b>	<b>16-18</b>

## Proposed Cold Neutron Instruments

Plan view through the fuel center of the reactor core



# Proposed Thermal Neutron Instruments



**View of Potential Thermal Instruments**

Instrument Type	Abbreviation
Prompt Gamma Neutron Activation Analysis	PGNAA
Neutron Microscope	Imaging
High-Resolution powder diffractometer	D
Triple Axis Spectrometer	3X
Ultra-Small Angle Neutron Scattering	USANS
High Throughput Fast Powder Diffractometer	D
White Beam Engineering Diffractometer (with CANDOR-type detector)	ENG
High Current Physics Experimental Position	PHYS

## Proposed Thermal Neutron Instruments

# Performance Comparison

Cold Source/ config	$J_{\text{tot}}$ (all $l$ ) ( $s^{-1}$ )	$J_{\text{tot}}$ ( $l \geq 4\text{\AA}$ ) ( $s^{-1}$ )
NBSR LH <sub>2</sub> Unit 2 (all cold guides)	$3.0 \times 10^{13}$	$6.3 \times 10^{12}$
NNS (6 cm $\times$ 15 cm)	$2.3 \times 10^{14}$	$5.8 \times 10^{13}$
Gain NNS/NBSR Unit2	7.5	9.2

Cold Source/ config	$J_{\text{tot}}$ (all $l$ ) ( $s^{-1}$ )	$J_{\text{tot}}$ ( $l \geq 4\text{\AA}$ ) ( $s^{-1}$ )
NBSR LH <sub>2</sub> Unit 2 (all cold guides)	$3.0 \times 10^{13}$	$6.3 \times 10^{12}$
NNS (6 cm $\times$ 20 cm)	$2.8 \times 10^{14}$	$7.0 \times 10^{13}$
Gain NNS/NBSR Unit2	9.1	11.1

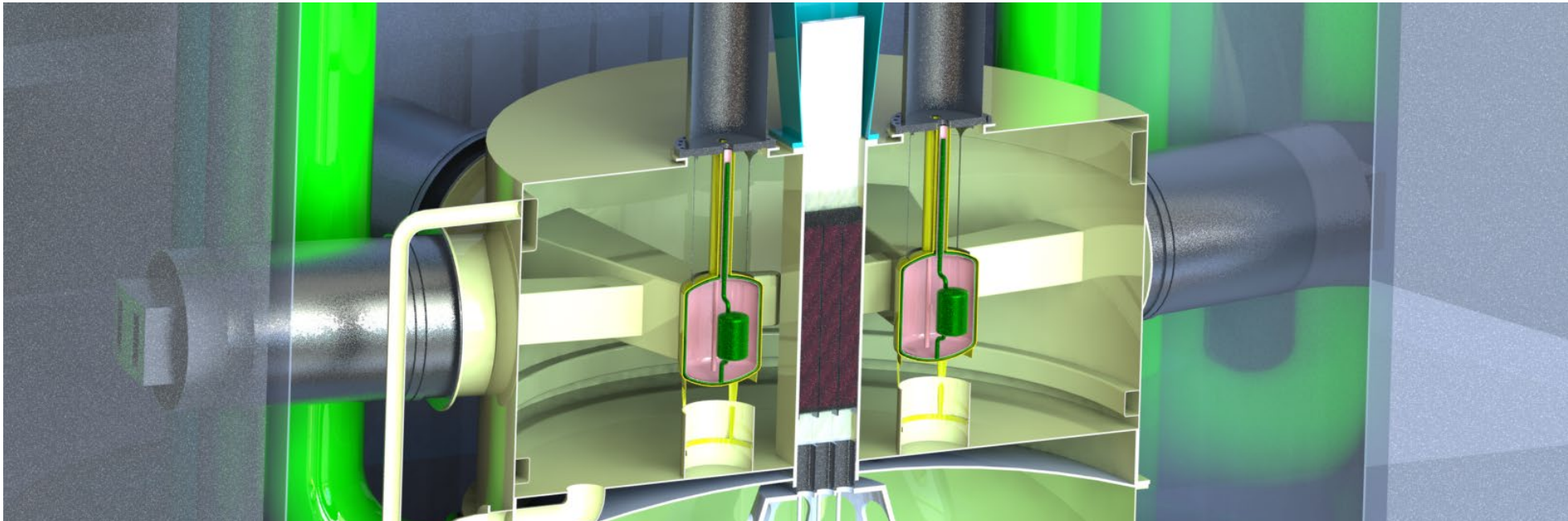
# Performance Comparison

- Peak unperturbed reflector thermal neutron flux
  - NBSR  $2 \times 10^{14} \text{ cm}^{-2} \text{ s}^{-1}$
  - NNS  $5 \times 10^{14} \text{ cm}^{-2} \text{ s}^{-1}$
- Total cold neutron ( $\lambda > 0.4 \text{ nm}$ ) current gain at guide entrances  **$\sim 10$  wrt NBSR LH<sub>2</sub> CNS**
- Gain at the instruments may be further enhanced
- Potential for a significant boost in the cold neutron experimental output
- Pool Type Reactor => simple maintenance
- Modular design for long term aging management



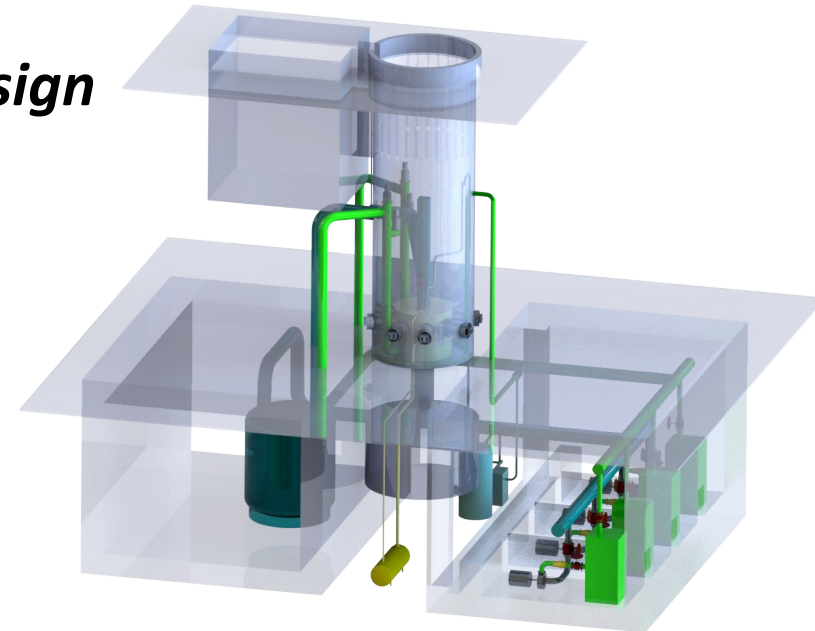
# Conclusions & Future Work

- NRC pre-engagement for licensing requirements
- Evaluate potential (qualified or in the process) fuel, U-10Mo, U<sub>3</sub>Si<sub>2</sub>, UZrH etc.
- Engagement with National Laboratories for non-proliferation, safety, realistic concept design, optimization, manufacturability and maintenance etc.
- Collaborate with Universities and Scientific Community to identify needs, resolve complex design issues etc.





- NIST Technical Note Series “Pre-conceptual Design Activities of the NIST Neutron Source”
  - Neutronics Safety Assessments
  - Layout of Cold & Thermal Neutron Instruments
  - Assessments of the Cold and Thermal Neutron Beams
  - Thermal-hydraulics Safety Assessments
  - ***NIST Neutron Source Pre-Conceptual Design***



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